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SUMMARY LISTING OF SUBCRITICAL MEASUREMENTS OF HETEROGENEOUS
WATER-URANIUM LATTICES MADE AT HANFORD

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CRITICAL MASS PHYSICS
PHYSICS AND INSTRUMENT RESEARCH
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RICHLAND, WASHINGTON

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INTRODUCTION

Exponential and critical approach type measurements have been made to determine the critical mass, material buckling, and in a few cases, the extrapolation length for the lattices. This report attempts to list all measurements on water-uranium heterogeneous lattices made to date at Hanford. All lattices were water moderated hexagonal arrays loaded with uranium of enrichments up to 3.1%.

Ratios given are volume ratios. The radii of loadings are calculated assuming cylindrical geometry by the following formula

$$R = \sqrt{\frac{nl^2\sqrt{3}}{2\pi}}$$

where n is the number of tubes and l is the lattice spacing (distance between fuel rod centers).

Plots of buckling and critical mass in spherical geometry vs H_2O/U volume ratios are included. The critical masses in spherical geometry are calculated from the measured bucklings assuming the same value of extrapolation length as used in calculating the bucklings.

The value of extrapolation length, λ , was taken from Brookhaven experiments except where noted. The curve of BNL data used was from BNL-C-7592. Values of λ for some of the 3.063 percent enriched uranium measurements were calculated by setting the bucklings for exponential and critical approach measurements equal. R_s , Vol_s and CM_s are estimated critical values for spherical geometry as calculated from measured values of buckling or cylindrical critical masses.

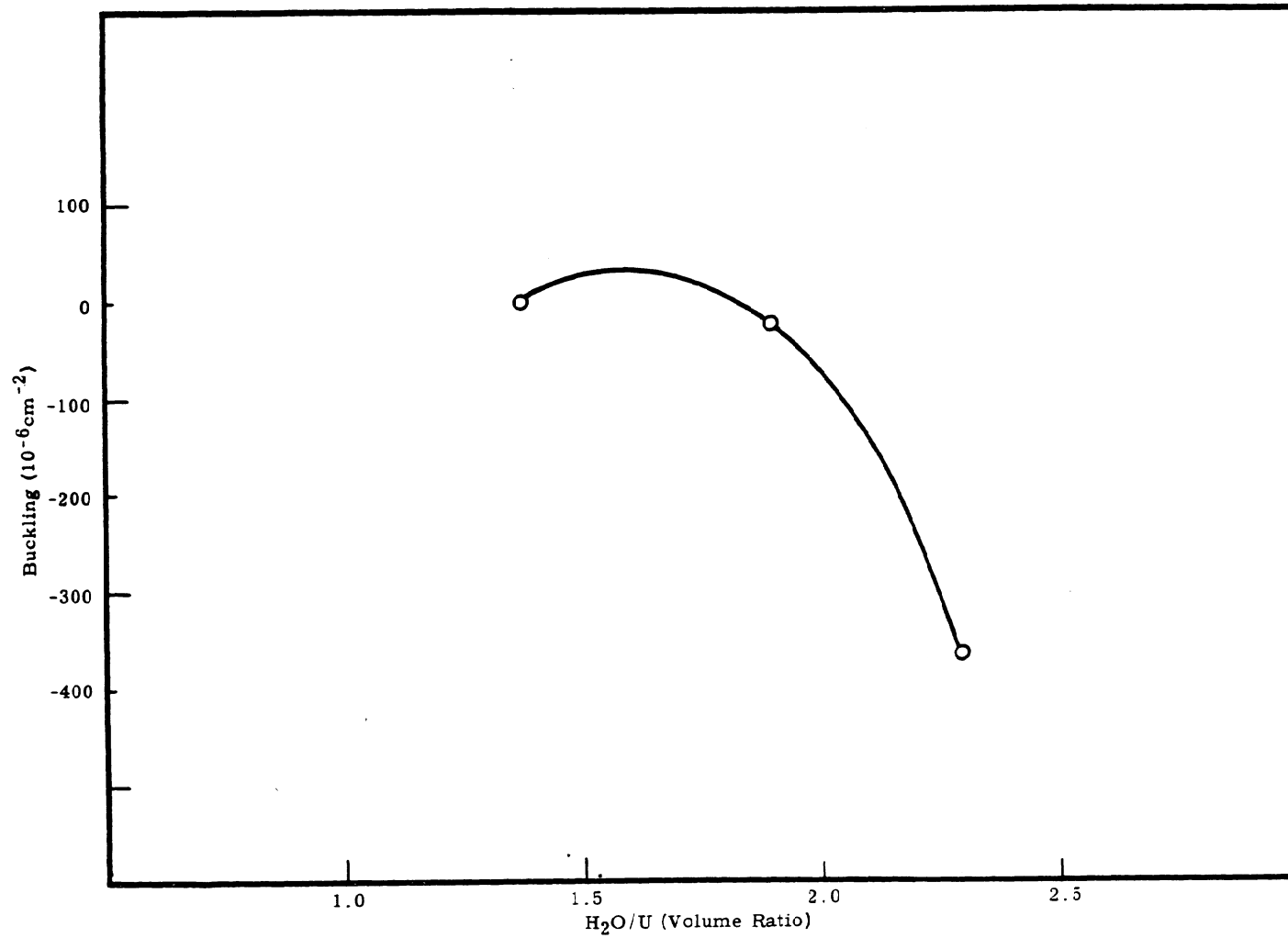
Standard error of the buckling is that calculated from the least square fit and does not include errors due to uncertainty in the value of extrapolation length.

References are listed so that more detailed information of each measurement can be readily obtained if desired.

EXPONENTIAL MEASUREMENTS OF NATURAL URANIUM

(0.925" dia.)

Date	Exp. No.	Lattice Spacing (inches)	Height, h (cm)	No. Tubes	H ₂ O/U Vol. Ratio	R _{eff} (cm)	b ₁₁ (cm)	λ (cm)	Buckling 10 ⁻⁶ cm ⁻²	Std. Error
5-60	108	1.400"	101.60	85	1.37	17.21	10.30	7.56	0	±55
3-60	102	1.500"	"	"	1.89	18.44	10.63	7.16	-24	±61
3-60	103	1.600"	"	"	2.29	19.67	10.83	6.96	-367	±40



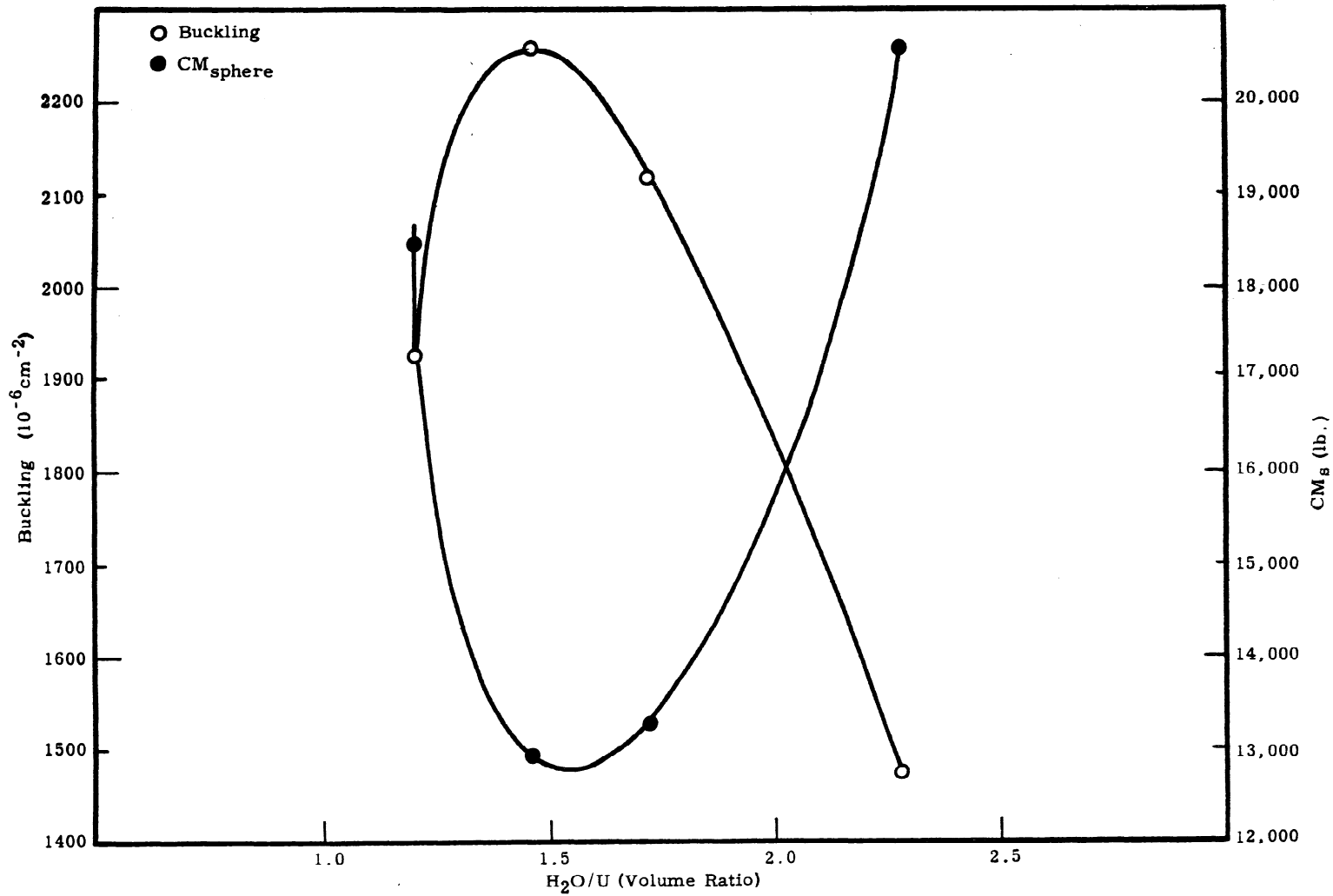
Buckling vs. $\text{H}_2\text{O}/\text{U}$ Ratio
(Natural Uranium, 0.925" Dia.)

EXPONENTIAL MEASUREMENTS OF 0.95 PER CENT ENRICHED URANIUM^{1,2}

Iron and Aluminum Encased 1.336" Diameter, 42" Length

Date	Exp. No.	Lattice Spacing (inches)	No. Tubes	Al/U (vol)	H ₂ O/U (vol)	R _{eff} (cm)	b ₁₁ (cm)	λ (cm)	Buckling (10 ⁻⁶ cm ⁻²)	Std. Error	R _s (cm)	Vol _s (liters)	CM _s (lbs)
8-58	70	2.00	55	0.159	1.20	19.78	13.21	7.70	1924	± 40	63.92	1,094	18,450
4-58	63	2.10	55	0.159	1.46	20.77	14.14	7.46	2257	± 32	58.68	846	12,943
4-58	62	2.20	55	0.159	1.72	21.76	14.51	7.27	2115	± 22	61.05	953	13,279
4-58	64	2.40	55	0.159	2.28	23.74	14.65	6.97	1474	± 34	74.86	1,758	20,582
Fe/U													
8-58	71	2.00	55	0.160	1.19	19.78	11.22	7.72	- 303	± 52	-	-	-
5-58	65	2.10	55	"	1.44	20.77	11.89	7.48	169	± 75	-	-	-
5-58	66	2.20	55	"	1.70	21.76	12.03	7.28	- 52	± 69	-	-	-
5-58	67	2.40	55	"	2.27	23.74	12.01	6.98	- 800	± 71	-	-	-

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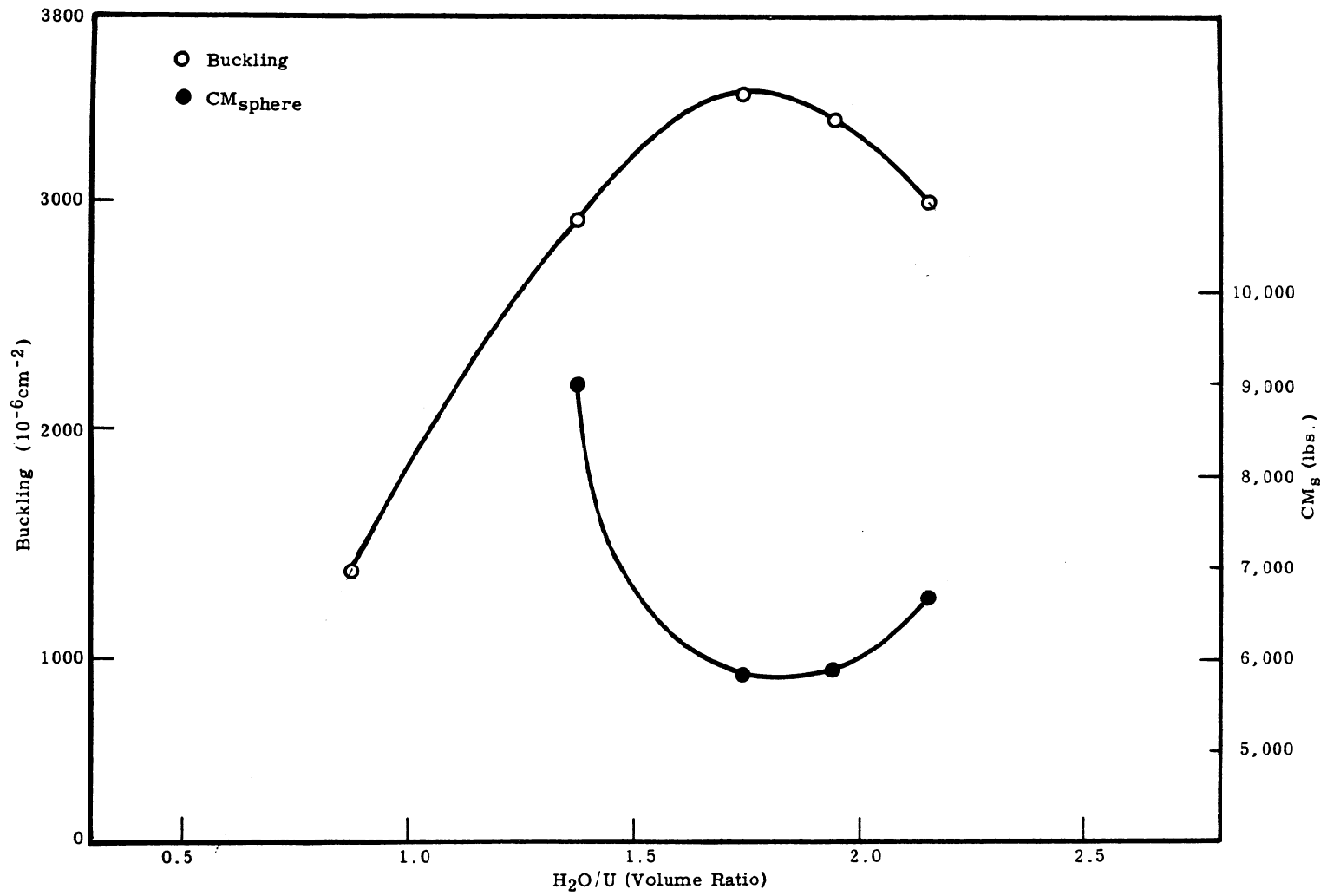


Buckling and Critical Mass vs. H₂O/U Ratio
 (.95% U-235 - 1.336" Diameter)

EXPONENTIAL MEASUREMENTS OF 1.007 PERCENT ENRICHED URANIUM^{3,4,5,6}
.925" DIAMETER, 44" LENGTH

Date	Exp. No.	Lattice Spacing (inches)	No. Tubes	Al/U (vol)	H ₂ O/U (vol)	R _{eff} (cm)	b ₁₁ (cm)	λ (cm)	Buckling (10 ⁻⁶ cm ⁻²)	Std. Error	R _g (cm)	Vols. (liters)	CM _g (lbs)
11-57	37	1.26	85	0.157	.87	15.49	10.58	8.20	1377	±42	76.47	1,873	38,146
	Project												
10-55	Bath	1.40	89	"	1.37	17.61	12.68	7.56	2903	±16	50.75	548	9,018
10-55	"	1.50	89	"	1.74	18.87	14.14	7.25	3470	±19	46.08	410	5,889
10-55	"	1.55	91	"	1.94	19.72	14.65	7.14	3357	±25	47.08	437	5,876
10-55	"	1.60	89	"	2.15	20.13	14.38	7.02	3012	±21	50.22	531	6,698

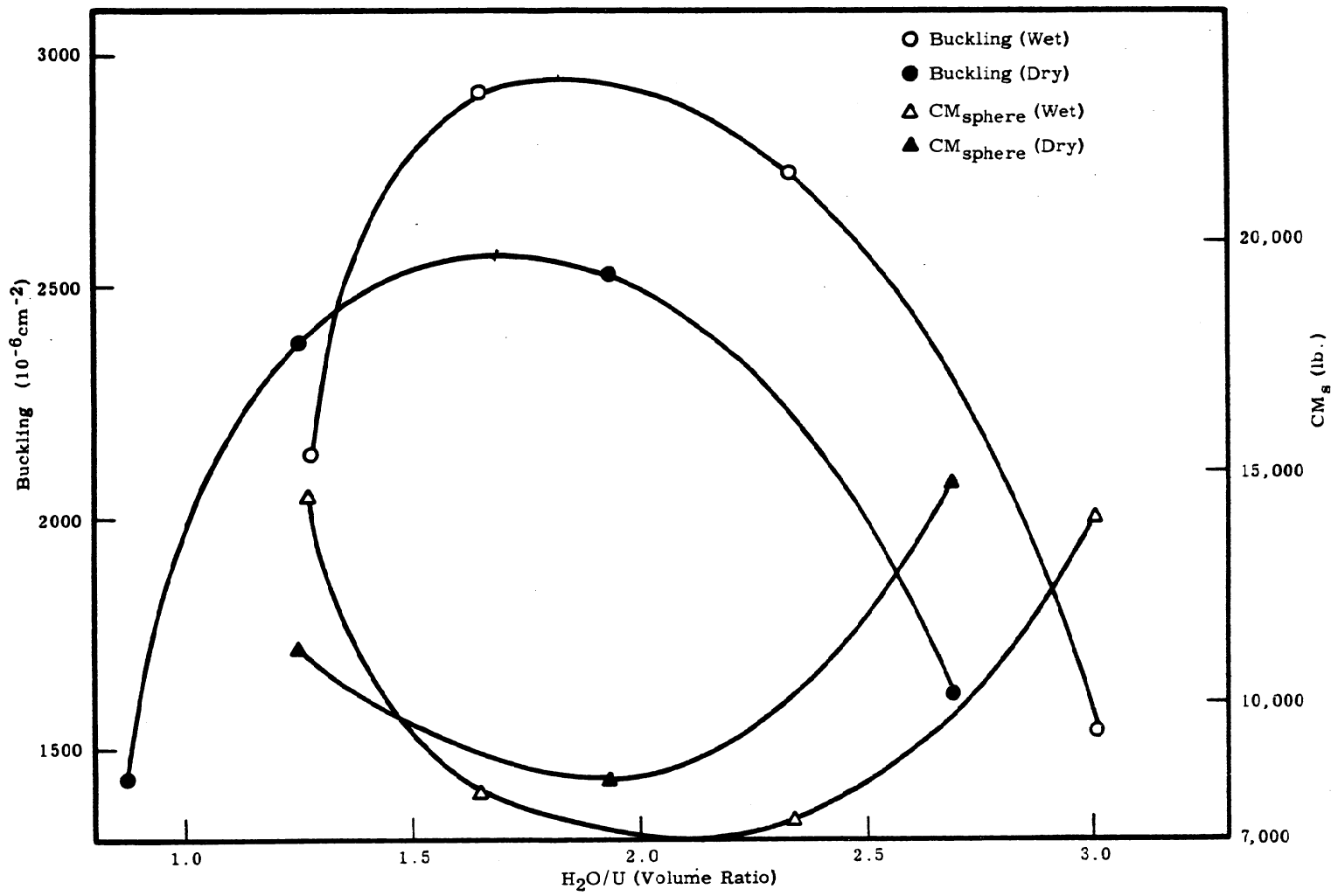
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Buckling and Critical Mass vs. H₂O/U Ratio
 (1.007 % U-235 - 0.925" Diam.)

EXPONENTIAL MEASUREMENTS OF 1.007 PERCENT ENRICHED URANIUM⁷
1.66" O.D.; 0.94" I.D., 44" LENGTH

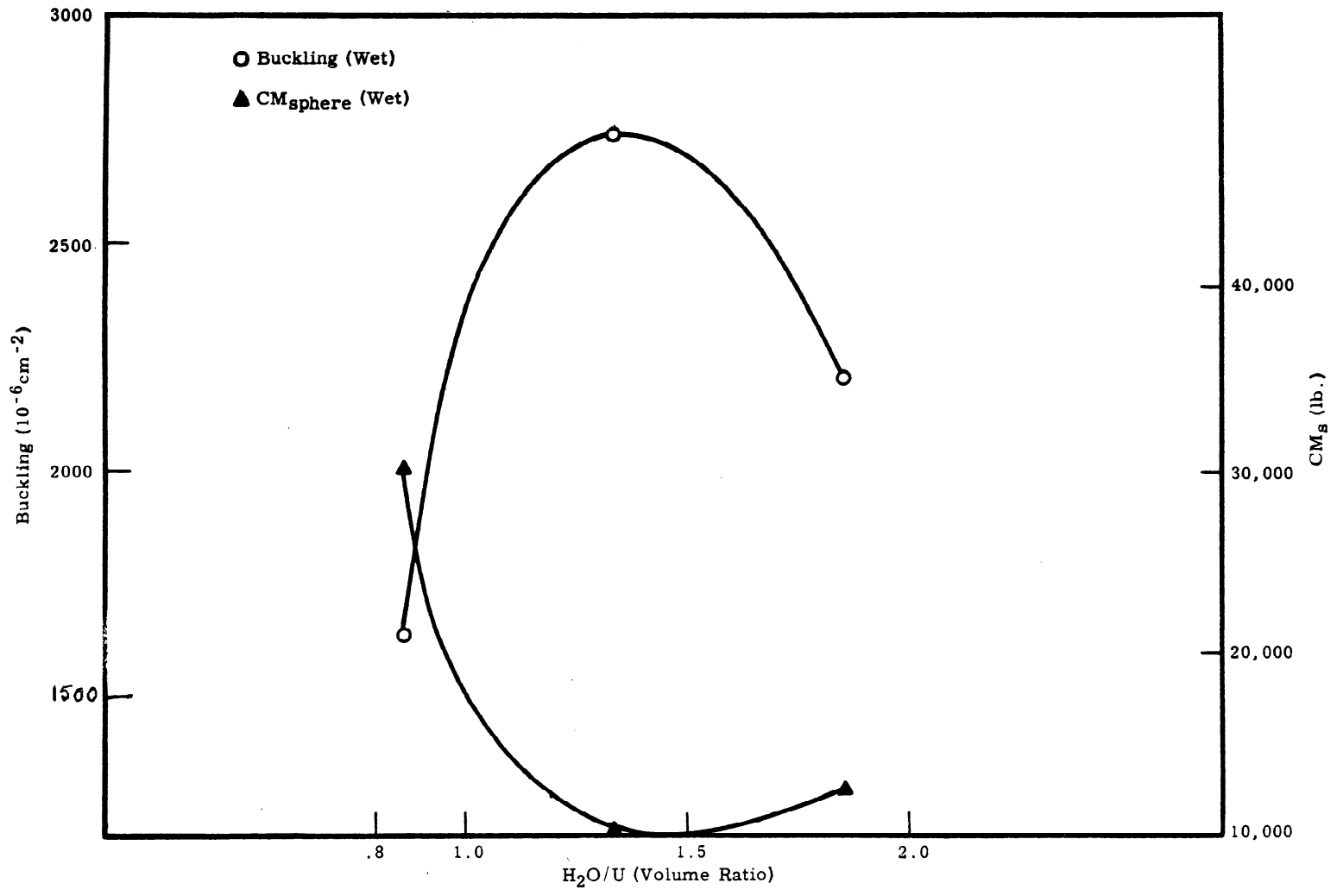
Date	Exp. No.	Lattice Spacing (inches)	No. Tubes	Fuel Core Cond.	Al/U (vol)	H ₂ O/U (vol)	R _{eff} (cm)	b ₁₁ (cm)	λ (cm)	Buckling (10 ⁻⁶ cm ⁻²)	Std. Error	R _s (cm)	Vol _s (liters)	CM _s (lbs.)
8-56	31-5	2.05	85	Dry	0.163	.873	25.20	16.36	8.25	1434	±42	74.711	1,747	29,348
8-56	31-4	2.05	"	Wet	"	1.272	25.20	17.61	7.66	2133	±9	60.365	921	15,480
7-56	31-3	2.20	"	Dry	"	1.248	27.05	20.33	7.67	2378	±6	56.760	766	11,195
7-56	31-2	2.20	"	Wet	"	1.648	27.05	22.53	7.34	2920	±15	50.795	549	8,023
9-56	31-9	2.45	"	Dry	"	1.933	30.12	24.72	7.14	2529	±13	55.331	710	8,361
9-56	31-8	2.45	"	Wet	"	2.333	30.12	26.12	6.94	2745	±14	53.027	625	7,360
8-56	31-6	2.70	"	Dry	"	2.692	33.20	22.38	6.80	1619	±20	71.275	1,517	14,717
8-56	31-7	2.70	"	Wet	"	3.091	33.20	21.79	6.68	1531	±5	73.602	1,670	14,036



Buckling and Critical Mass vs. H_2O/U Ratio
 (1.007% U-235 - 1.66" O.D. - 0.94" I.D.)

EXPONENTIAL MEASUREMENTS OF 1.007 PERCENT ENRICHED URANIUM^{3,4}
1.66" DIAMETER, 44" LENGTH

Date	Exp. No.	Lattice Spacing (inches)	No. Tubes	Al/U (vol)	H ₂ O/U (vol)	R _{eff} (cm)	b ₁₁ (cm)	λ (cm)	Buckling (10 ⁻⁶ cm ⁻²)	Std. Error	R _s (cm)	Vol. (liters)	CM _s (lbs)
10-55	Project Bath	2.20	87	.069	0.86	27.37	18.48	8.22	1639	±36	69.39	1,399	30,104
10-55	Tub	2.45	"	"	1.33	30.48	28.34	7.59	2747	±7	52.36	601	10,428
10-55	"	2.70	"	"	1.85	33.59	28.00	7.18	2205	±11	59.73	893	12,749

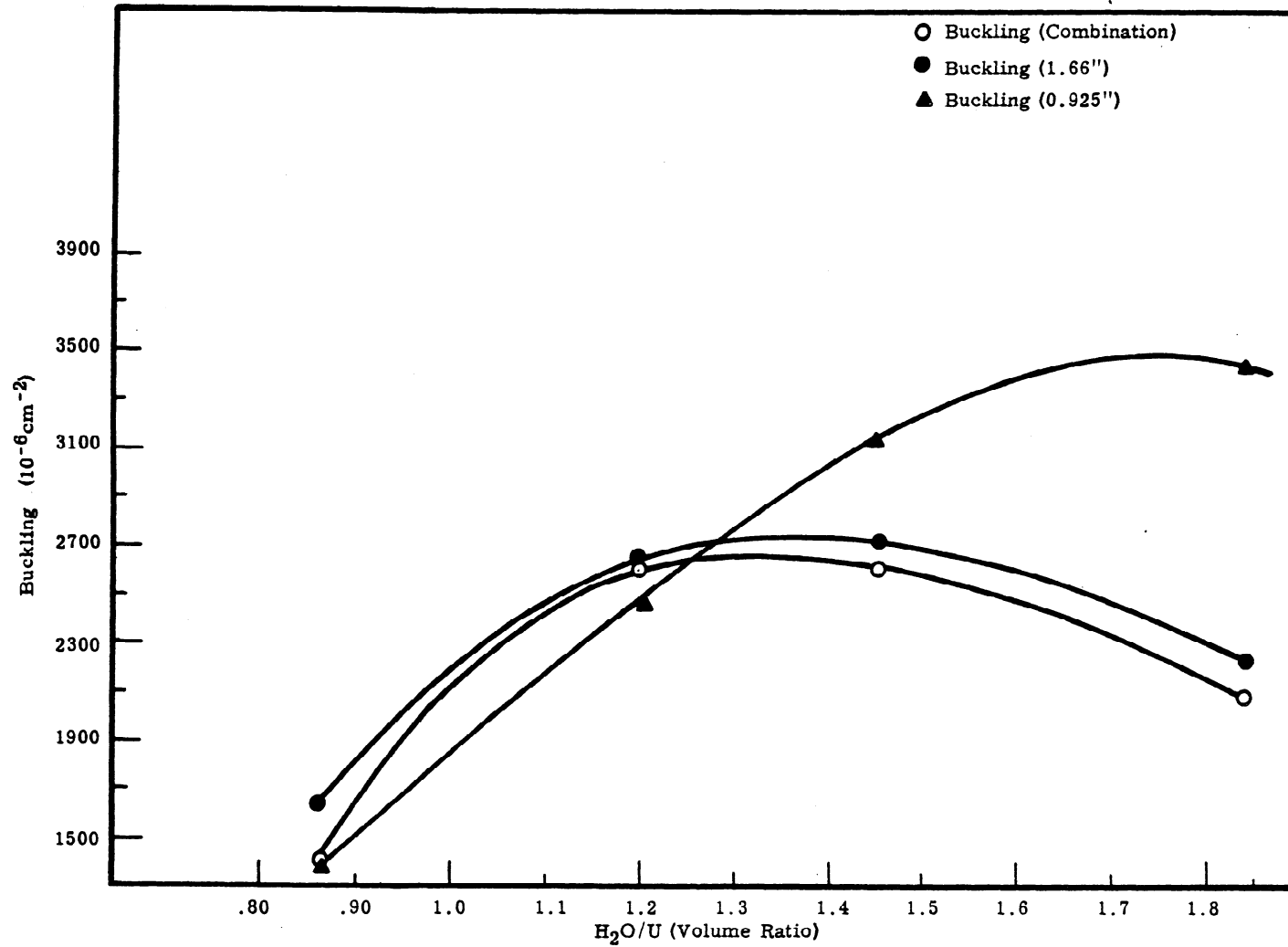


Buckling and Critical Mass vs. H₂O/U Ratio
 (1.007% U-235 - 1.66" Diameter)

HW-65552

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EXPONENTIAL MEASUREMENTS OF 1.007 PERCENT ENRICHED URANIUM
COMBINATION LATTICES OF 1.66 INCH AND 0.925 INCH DIAMETERS, 44 INCH LENGTH

Date	Exp. No.	Lattice Spacing (inches)	H ₂ O/U (vol)	No. Tubes		b ₁₁ (cm)	λ (cm)	Combination Buckling (10 ⁻⁶ cm ⁻²)	1.66 inch Buckling	.925 inch Buckling
				.925	1.66					
5-21-58	68	1.80	0.86	44	43	14.45	8.19	1396	1639	1377
2-6-58	49a	1.95	1.20	44	41	17.60	7.25	2524	2640	2440
1-17-58	46	2.05	1.45	44	41	18.23	6.87	2606	2700	3130
2-3-58	48	2.20	1.84	44	41	18.18	6.61	2078	2230	3440



Buckling vs. H₂O/U Ratio
 (1.007% U-235 - Combination Lattices of 1.66" & 0.925"
 Diameters)

BUCKLING MEASUREMENTS FOR FUEL ELEMENTS IN A RANDOM^{6,8,9}
ARRAY VS A UNIFORM ARRAY

A series of buckling measurements had previously been taken with random arrays of fuel elements to see if such configurations would have bucklings significantly less than those resulting from a uniform distribution of fuel elements. The results of these measurements, together with the error limits, are summarized in the following table.

The average value of the measured bucklings are less in all three cases for the random arrays, but, in view of the 95 percent confidence limits predicted from the t distribution, it is not advisable to increase the safe critical mass limits for the fuel elements in a random distribution over the critical mass based on the most reactive uniform rod lattice. Large errors are inherent in the reduction of data from this type of exponential experiment because of local perturbations in the neutron flux caused by local variations in the H₂O/U volume ratio. The error limits, predicted by student t distribution, were determined by personnel of the Research and Synthesis Operation.

SUMMARY OF BUCKLING VALUES FOR FUEL
ELEMENTS IN RANDOM ARRAYS

Bare Fuel Elements

1.007% Enriched

0.925-inch Diameter

4 Inches in Length

Exp. No.	H ₂ O/U (by volume)	Measured Buckling for Random Array	Difference from Average	95% Confidence Limits Predicted From t Distribution	Corresponding Buckling of Fuel Elements in Uniform Array (Hexagonal Lattice)
32	0.86	-113 x 10 ⁻⁶ cm ⁻²	-231 x 10 ⁻⁶ cm ⁻²	(Upper, Lower Limit of Buckling)	1377 x 10 ⁻⁶ cm ⁻²
33	0.87	-727	-845		
35	0.86	744	+626		
36	0.86	568	+450	1166.05 x 10 ⁻⁶ cm ⁻²	
	Avg. 0.86	Avg. 118 x 10 ⁻⁶ cm ⁻²		-979 x 10 ⁻⁶ cm ⁻²	

Bare Fuel Elements

1.007% Enriched

1.66 Inches O.D., 0.94 Inches I.D.

4 Inches in Length

38	1.49	2139 x 10 ⁻⁶ cm ⁻²	+90 x 10 ⁻⁶ cm ⁻²	2651 x 10 ⁻⁶ cm ⁻² 1388 x 10 ⁻⁶ cm ⁻²	3075 x 10 ⁻⁶ cm ⁻²
39	1.51	2149	+100		
40	1.46	2428	+379		
41	1.47	1418	-569		
	Avg. 1.48	Avg. 2049 x 10 ⁻⁶ cm ⁻²			

Aluminum Clad Fuel Elements

1.44% Enriched

Clad Dimensions

1.47-Inch O.D., 0.37-Inch I.D.

8.6 Inches in Length

Bare Dimensions

1.37-Inch O.D., 0.48-inch I.D.

8 Inches in Length

42	1.78	3271 x 10 ⁻⁶ cm ⁻²	-96 x 10 ⁻⁶ cm ⁻²	3901 x 10 ⁻⁶ cm ⁻² 2798 x 10 ⁻⁶ cm ⁻²	5850 x 10 ⁻⁶ cm ⁻²
43	1.78	3225	-142		
44	1.80	3785	+418		
45	1.74	3777	+410		
50	1.93	2775	-592		
	Avg. 1.81	Avg. 3367 x 10 ⁻⁶ cm ⁻²			

EXPONENTIAL MEASUREMENTS OF 1.007 PERCENT ENRICHED URANIUM

UNH (0.68%) REFLECTED ASSEMBLIES - 0.925 INCH DIAMETER
(1.5 inch lattice spacing - water moderator)

<u>Date</u>	<u>Experiment</u>	<u>.68% U-235 - Reflector</u>	<u>Relaxation Length</u> (cm)	<u>λ(cm)</u>
10-55	Bathtub	536 gm/l	14.7	8.10 ± .2
10-55	"	188 gm/l	14.3	7.64
10-55	"	H ₂ O	13.9	7.25 - BNL data

EXPONENTIAL MEASUREMENTS OF 1.007 PERCENT ENRICHED URANIUM⁵
UW (1.007%) REFLECTED ASSEMBLIES - 0.925 INCH DIAMETER

Exp. No.	Lattice Spacing Inches	Moderator Solution gms Uranium per Liter of Water	Reflector Solution gms Uranium per Liter of Water	Reflector Thickness cm	Relaxation Length (b_1) cm	Error ^(b) in b_1 (S_{b_1}) cm	Nitric Acid Content of Uranyl Nitrate gm/Liter	Exponential Assembly Conditions
3	1.4	zero	502	43.8	13.02	0.03	50.3	1/16-inch stainless steel lattice tank
12	1.4	"	406	43.8	12.81	0.03	54.8	" "
13	1.4	"	zero	43.8	12.00	0.04	--	" "
14	1.4	"	zero	43.8	12.37	0.03	--	No stainless steel tank
4	1.5	"	495 ^(a)	42.5	14.22	0.04	50	1/16-inch stainless steel lattice tank
5	1.5	"	zero	42.5	13.07	0.03	--	" "
6	1.5	"	486	12.0	14.11	0.04	50.3	" "
7	1.5	"	490	22.2	14.31	0.03	50.3	" "
8	1.5	"	406	22.2	14.05	0.04	41.0	" "
9	1.5	"	203	22.2	13.58	0.04	16.2	" "
10	1.5	"	203	42.5	13.36	0.05	37.6	" "
11	1.5	"	406	42.5	13.90	0.04	45.3	" "
21	1.5	"	179	12.0	13.44	0.05	34.7	" "

(a) Estimated from values for experiments 3 and 6.

(b) The degrees of freedom for all experiments are four.

Exp. No.	Lattice Spacing inches	Moderator Solution gms Uranium per Liter of Water	Reflector Solution gms Uranium per Liter of Water	Reflector Thickness cm	Relaxation Length (b_1) cm	Error ^(b) in b_1 (S_{b_1}) cm	Nitric Acid Content of Uranyl Nitrate gm/liter	Exponential Assembly Conditions
23	1.5	431	zero	12.0	11.19	0.03	18.7	1/16-inch stainless steel lattice tank
2	1.6	zero	526	41.3	15.20	0.08	61.4	" "
16	1.6	zero	405	41.3	14.92	0.02	47.8	" "
17	1.6	zero	zero	41.3	13.93	0.03	--	" "
18	1.6	zero	zero	41.3	13.57	0.04	--	3/16-inch stainless steel lattice tank
19	1.6	zero	zero	41.3	14.46	0.07	--	No stainless steel tank
22	1.6	zero	zero	41.3	14.44	0.06	--	" "

(b) The degrees of freedom for all experiments are four.

Cylindrical radii of lattice cores: 1.4-inch lattice 17.21 cm, 1.5-inch lattice 18.44 cm, 1.6-inch lattice 19.67 cm.

EFFECTIVE URANYL-NITRATE REFLECTOR SAVINGS
VS. URANIUM CONCENTRATION (a)

<u>Experiment Number</u>	<u>Reflector Concentration Grams Uranium Per Liter of Water</u>	<u>Reflector Thickness (cm)</u>	<u>Reflector^(b) Savings (cm)</u>
9	203	22.20	7.18
8	406	22.20	7.72
7	490	22.20	8.01
21	179	12.04	7.08
6	486	12.04	7.78
5	H ₂ O	47.52	6.58 ^(c)

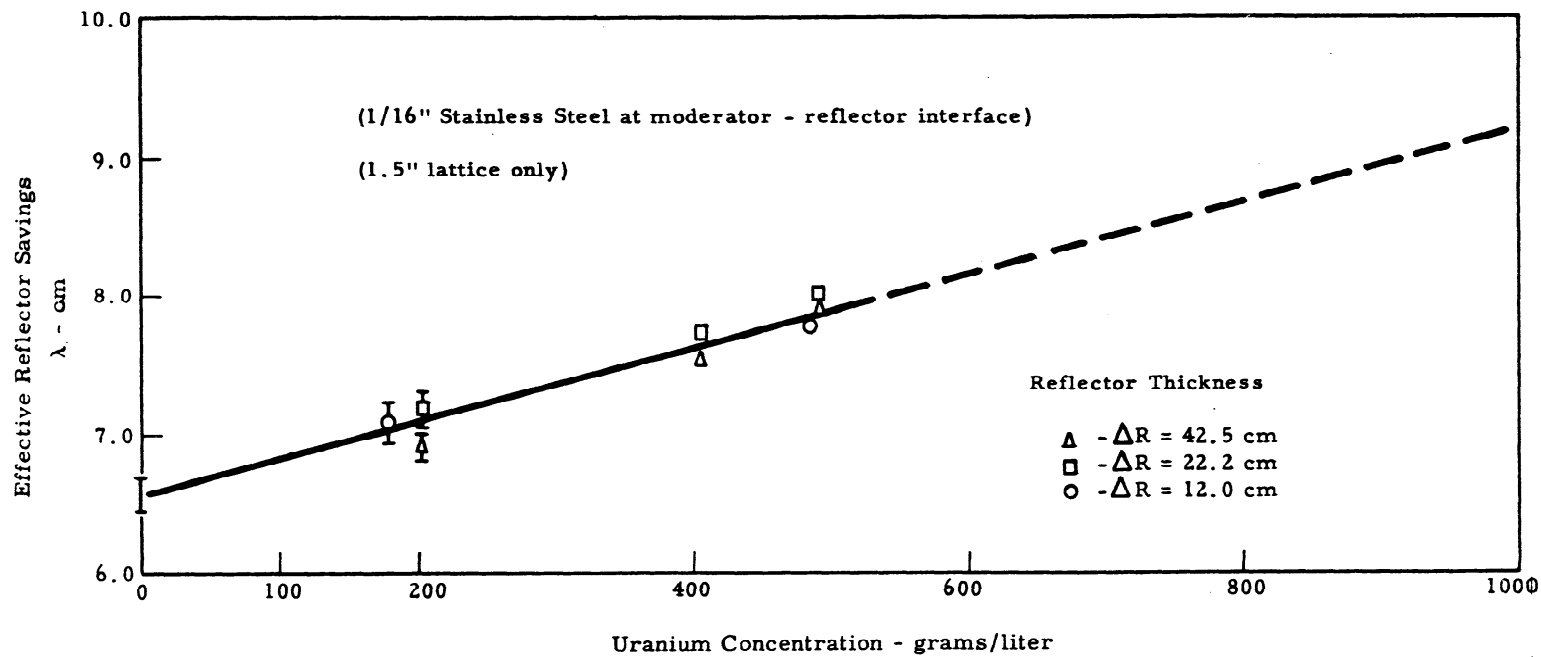
(a) Data are plotted in figure following.

Lattice spacing for all experiments is 1.5 inches and

$$B_m^2 = 3386 \times 10^{-6} \text{cm}^{-2}$$

(b) Reflector savings uncertainties 0.1 to 0.25 cm.

(c) Same for all three reflector thicknesses.



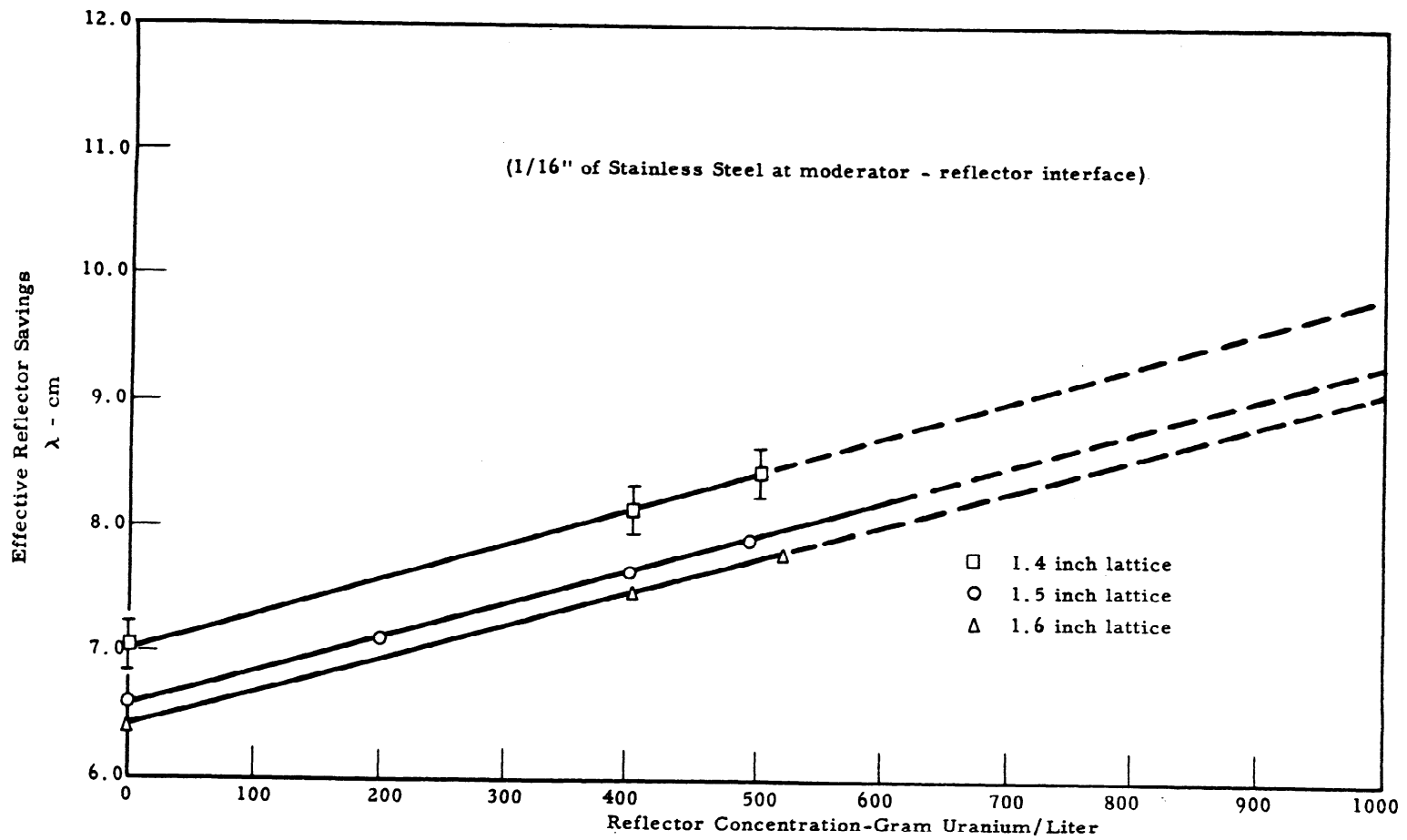
Effective Uranyl-Nitrate Reflector Savings vs Uranium Concentration
for Reflector Thicknesses 12.0, 22.2, 42.5 cm

EFFECTIVE^(a) URANYL NITRATE REFLECTOR SAVINGS
VS. URANIUM CONCENTRATION^(b)

<u>Experiment Number</u>	<u>Lattice Spacing Inches</u>	<u>Reflector Concentration Gram of Uranium per Liter of Water</u>	<u>Reflector Thickness (cm)</u>	<u>$\lambda + R$ (cm)</u>	<u>λ_{UNH}</u>	<u>Miscellaneous</u>
13	1.4	zero	43.75	24.24	7.03	$B_m^2 = 2869 \mu b$
12	1.4	406	43.75	25.36	8.15	R = 17.21 cm
3	1.4	502	43.75	25.64	8.43	
5	1.5	zero	42.52	25.02	6.58	$B_m^2 = 3386 \mu b$
10	1.5	203	42.52	25.37	6.93	R = 18.44 cm
11	1.5	406	42.52	25.99	7.55	
4	1.5	495	42.52	26.35	7.91	
17	1.6	zero	41.29	26.11	6.43	$B_m^2 = 3336 \mu b$
16	1.6	405	41.29	27.19	7.52	R = 19.67 cm
2	1.6	526	41.29	27.48	7.81	

(a) These data are plotted in figure following.

(b) These are effective values, due to the stainless steel at moderator-reflector interface.



Effective Uranyl Nitrate Reflector Savings vs Uranium Concentration

EXPONENTIAL MEASUREMENTS OF 1.007 PERCENT ENRICHED URANIUM^{3,4}
COMPARISON OF AIR AND WATER REFLECTOR

(1.55" lattice, .925" diameter, 44" length, H₂O/U = 1.94)

<u>Date</u>	<u>Condition</u>	<u>b₁₁ (cm)</u>	<u>λ (cm)</u>
11-55	H ₂ O Reflector - 1/8" Al Tank	14.6	7.14
11-55	H ₂ O Reflector - Cd cover tank	12.7	
11-55	No Reflector - Cd cover tank	11.6	3.4 ± 0.2

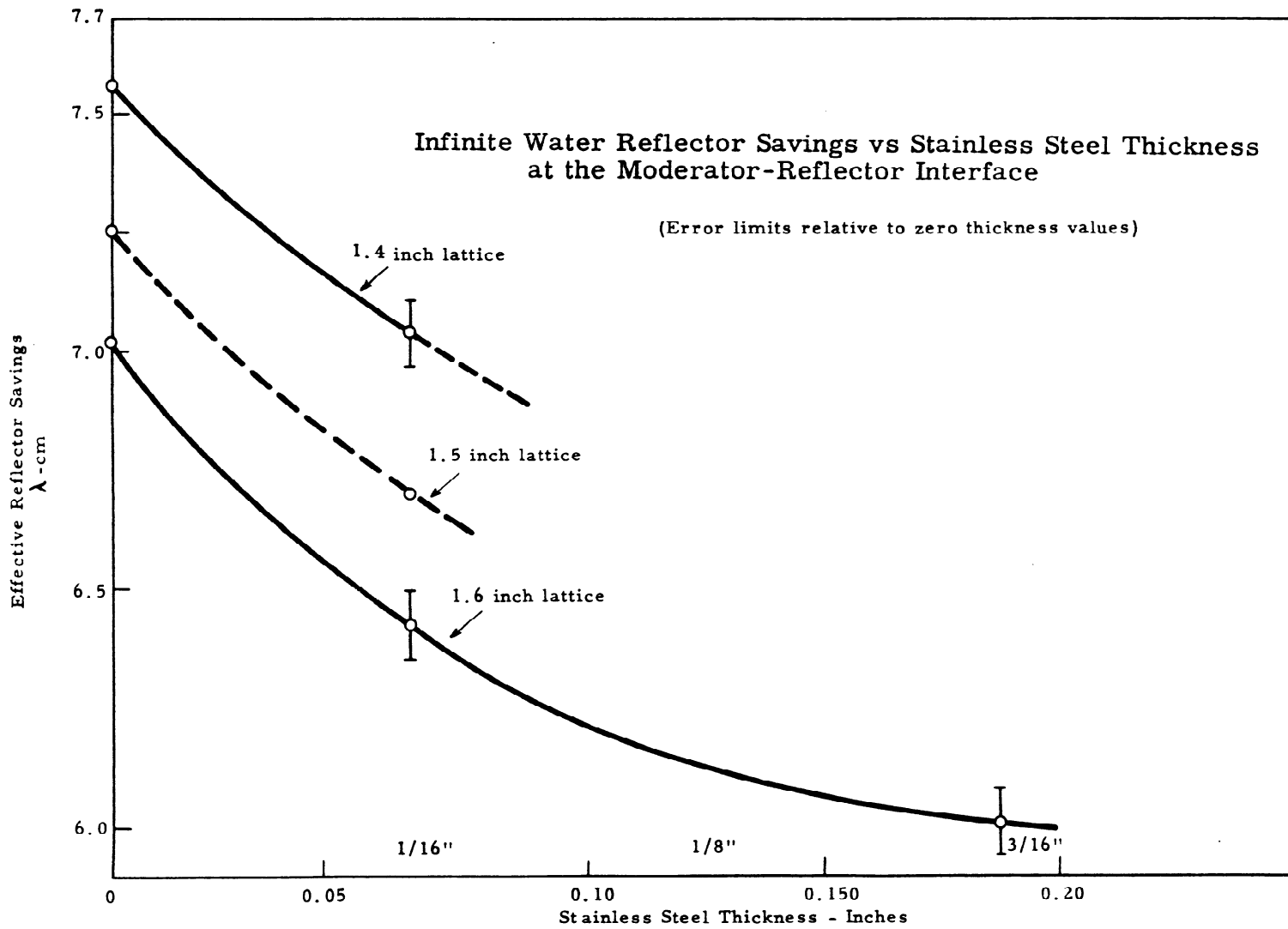
INFINITE WATER REFLECTOR SAVINGS VS. STAINLESS STEEL
THICKNESS AT THE MODERATOR - REFLECTOR INTERFACE

<u>Experiment Number</u>	<u>Lattice Spacing (inches)</u>	<u>Stainless Steel Thickness (inches)</u>	<u>Reflector Savings λ (cm)</u>	<u>Standard Error in S_{λ} (cm)</u>
14	1.4	0	7.56	0.05 to 0.25 ^(a)
13	1.4	1/16	7.04	0.1 to 0.25
19 and 22	1.6	0	7.02	0.05 to 0.25 ^(a)
17	1.6	1/16	6.43	0.1 to 0.25
18	1.6	3/16	6.01	0.1 to 0.25
x ^(b)	1.5	0	7.25	0.05 to 0.25 ^(a)
5	1.5	1/16	6.70 ^(c)	

(a) Zero thickness-Stainless steel reflector savings and errors obtained from BNL data.

(b) Experiment not done.

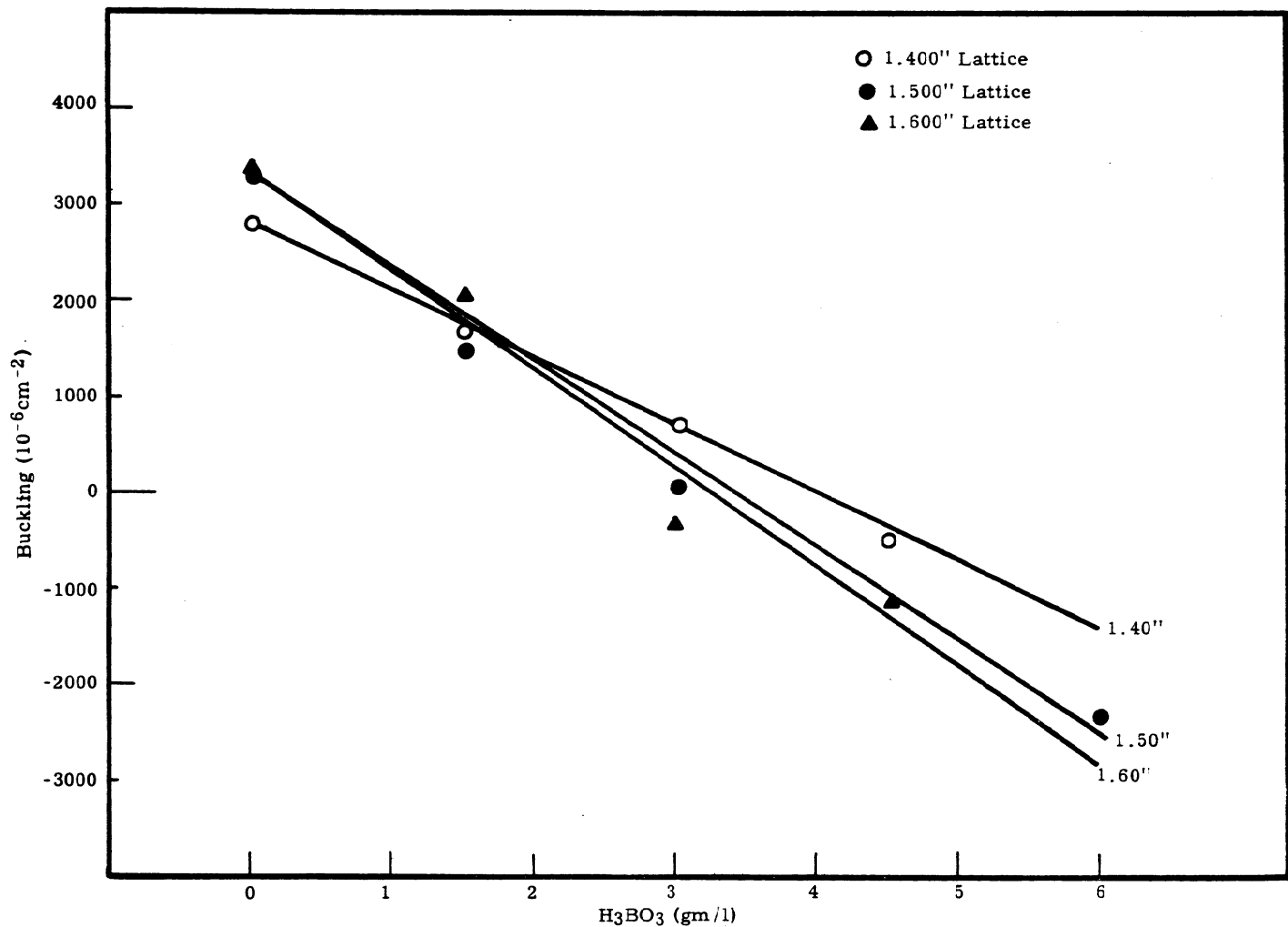
(c) Determined by linear interpolation between 1.4-inch and 1.6-inch lattices values.



EXPONENTIAL MEASUREMENT OF 1.007 PERCENT ENRICHED URANIUM WITH BORON POISONED MODERATOR
(0.925 inch Diameter, 44 inches Length)

Date	Exp. No.	Lattice Spacing (inches)	H ₂ O/U (volume)	Al/U (volume)	No. Tubes	R _{eff} (cm)	H ₃ BO ₃ (gm/l)	Boron (gm/l)	b ₁₁ (cm)	λ* (cm)	Buckling (10 ⁻⁶ cm ⁻²)	H ₃ BO ₃ at B ² = 0 (gm/l)
3-60	104	1.400	1.37	0.157	85	17.21	0	0	11.84	6.92	2794	
4-60	105	"	"	"	"	"	1.5	.26	10.99	"	1643	4.0 ± 0.3
4-60	106	"	"	"	"	"	3.0	.53	10.39	"	663	
4-60	107	"	"	"	"	"	4.5	.79	9.90	"	-269	
9-59	85, 86	1.500	1.74	"	"	18.44	0	0	13.03	6.65	3294	
9-59	89	"	"	"	"	"	1.5	.26	11.41	"	1502	3.4 ± 0.5
9-59	87	"	"	"	"	"	3.0	.53	10.49	"	97	
9-59	88	"	"	"	"	"	6.0	1.06	9.33	"	-2303	
12-59	97	1.600	2.15	"	"	19.67	0	0	13.94	6.43	3341	
2-60	98	"	"	"	"	"	1.5	.26	12.47	"	2061	3.2 ± 0.5
2-60	99	"	"	"	"	"	3.0	.53	10.66	"	-313	
3-60	100	"	"	"	"	"	4.5	.79	10.18	"	-1162	

*BNL-C-7592 values multiply by 0.916 to correct for 1/16" small stainless steel tank at the moderator-reflector interface.

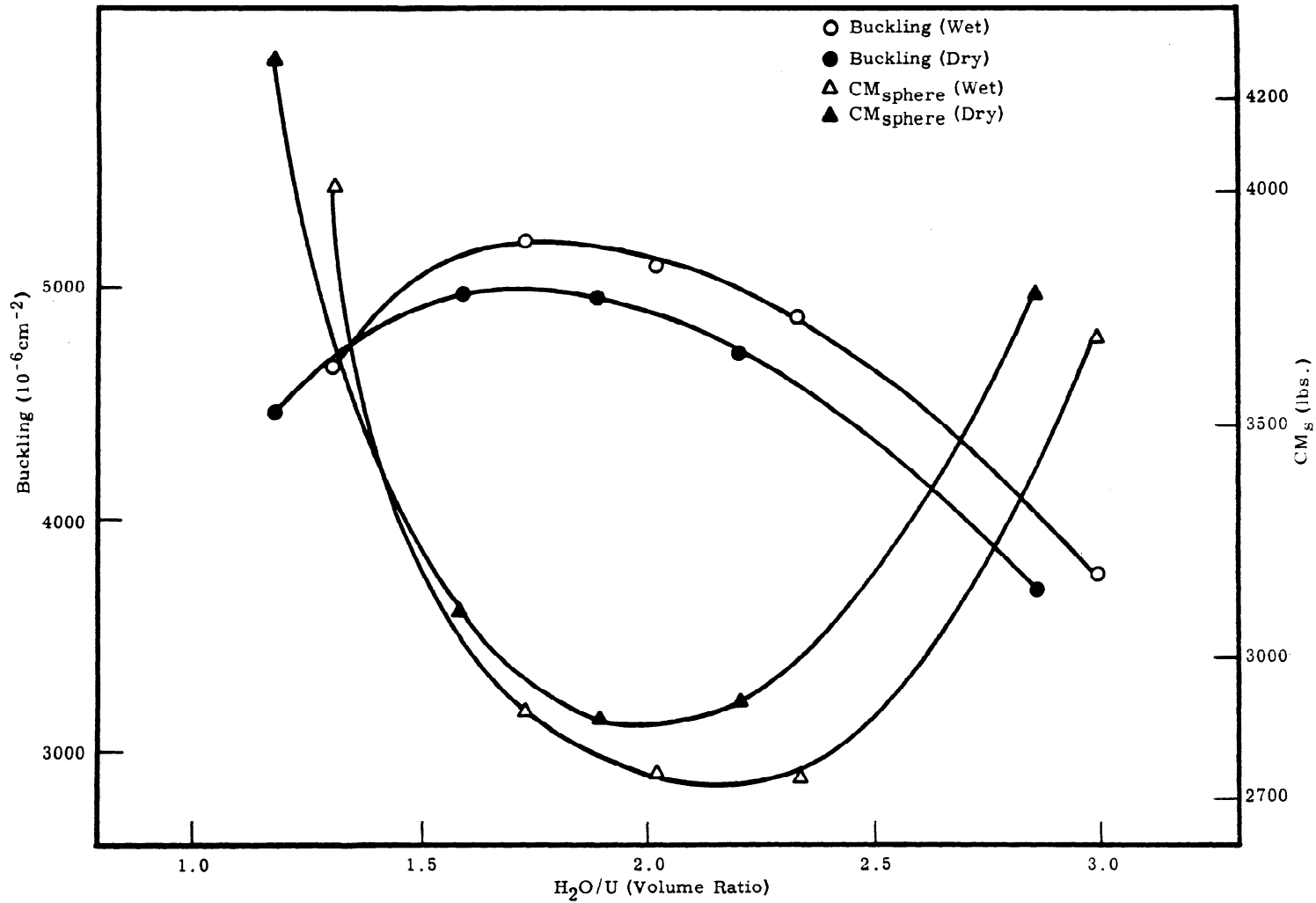


Buckling of 1.007 Per Cent Enriched Uranium vs. H_3BO_3 (gm/l) in Moderator

EXPONENTIAL MEASUREMENTS OF 1.25 PER CENT ENRICHED URANIUM¹⁰

1.336" O.D. 0.500" I.D.

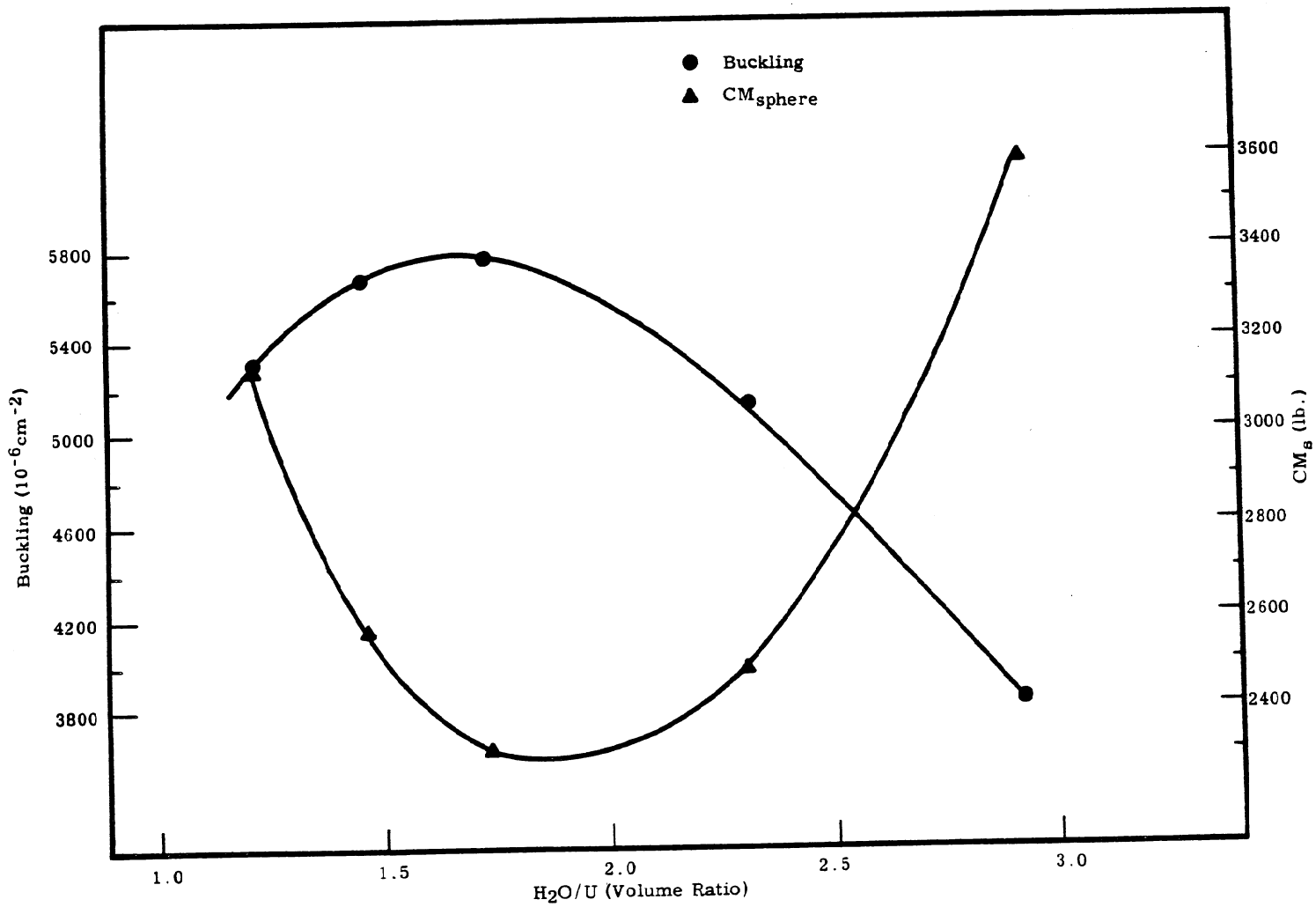
Date	Exp. No.	Lattice Spacing (inches)	h (cm)	No. Tubes	Fuel Core Cond.	Al/U (vol)	H ₂ O/U (vol)	R _{eff} (cm)	b ₁₁ (cm)	λ (cm)	Buckling (10 ⁻⁶ cm ⁻²)	Std. Error	R _s (cm)	Vol _s (Liters)	CM _s (lbs)
3-59	84	1.85	114.14	55	Wet	.125	1.31	18.30	15.91	7.63	4650	± 19	38.44	238	4004
4-59	85	"	"	"	"	.125	1.31	"	15.90	"	4645	± 19	38.47	238	4011
3-59	83	"	"	"	Dry	.100	1.18	"	15.70	7.77	4450	± 35	39.33	255	4286
1-59	78	2.00	"	"	Wet	.125	1.73	19.78	19.26	7.29	5196	± 14	36.29	200	2883
1-59	79	"	"	"	Dry	.100	1.59	"	18.66	7.38	4969	± 23	37.19	215	3101
12-58	74	2.10	101.60	"	Wet	.125	2.02	20.77	20.93	7.115	5152	± 22	36.65	206	2641
1-59	75	"	114.14	"	"	.125	2.02	"	20.66	7.115	5092	± 3	36.91	211	2750
1-59	80	"	"	"	Dry	.100	1.89	"	20.24	7.19	4957	± 9	37.43	220	2868
11-58	72	2.20	101.60	"	Wet	.125	2.33	21.76	21.05	6.96	4755	± 68	38.60	241	2810
1-59	76	"	114.14	"	"	.125	2.33	"	21.63	6.96	4874	± 13	38.04	231	2743
2-59	81	"	"	"	Dry	.100	2.20	"	20.97	7.02	4709	± 10	38.76	244	2902
11-58	73	2.40	101.60	"	Wet	.125	2.99	23.74	19.72	6.72	3661	± 9	45.20	387	3792
1-59	77	"	114.14	"	"	.125	2.99	"	20.10	6.72	3757	± 10	44.53	370	3698
3-59	82	"	"	"	Dry	.100	2.86	"	19.92	6.76	3696	± 10	44.92	380	3794



Buckling and Critical Mass vs. H_2O/U Ratio
 (1.25% U-235 - 1.336" O.D. -
 0.500" I.D.)

EXPONENTIAL MEASUREMENTS OF 1.44 PERCENT ENRICHED URANIUM⁷
1.336" diameter, 32" length

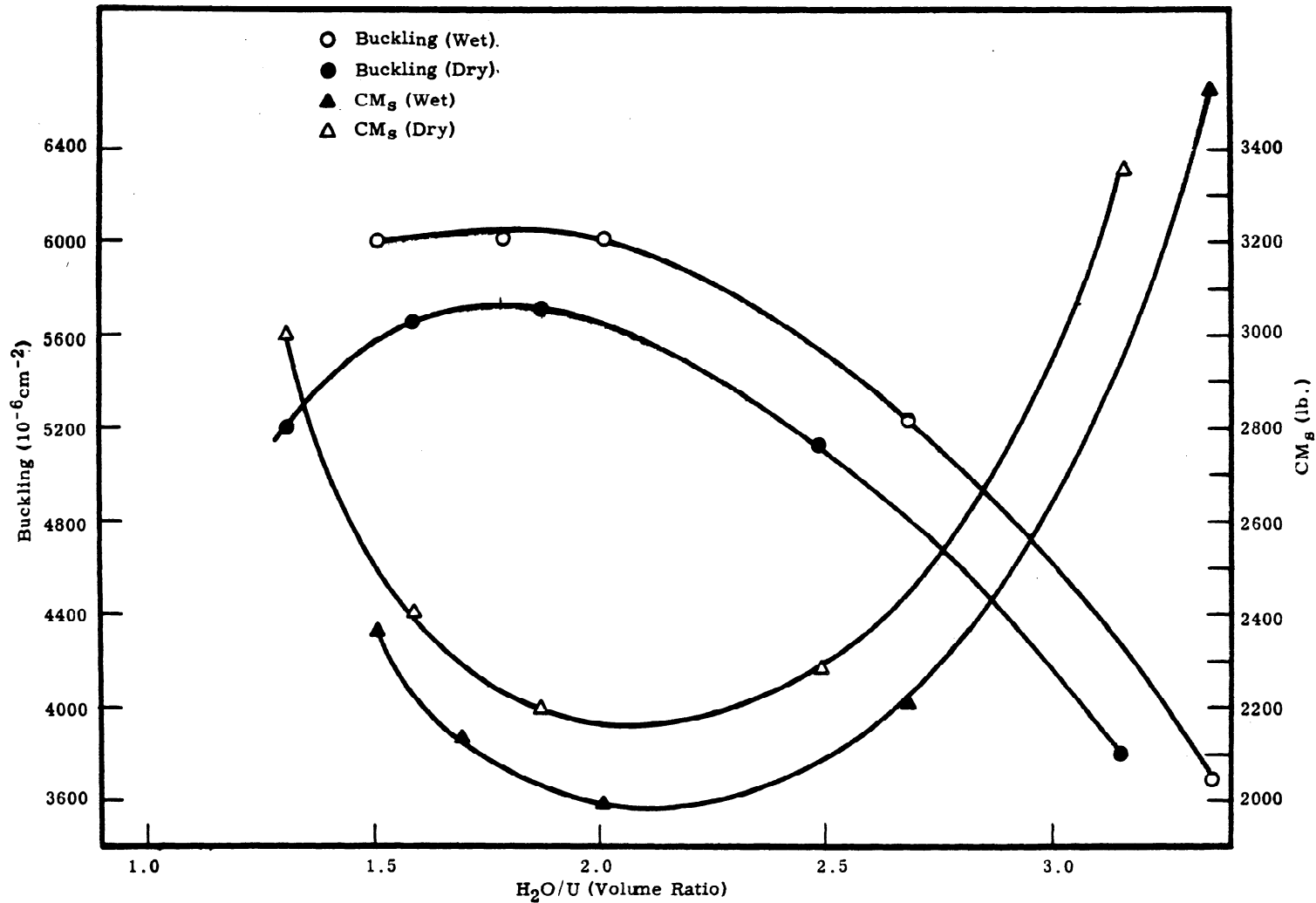
Date	Exp. No.	Lattice Spacing Inches	No. Tubes	Al/U (vol)	H ₂ O/U (vol)	R _{eff} (cm)	b ₁₁ (cm)	λ (cm)	Buckling (10 ⁻⁶ cm ⁻²)	Std. Error	R _g (cm)	Vol _g (Liters)	CM _g (lbs)
7-56	33-6	2.00	49	0.159	1.21	18.67	18.23	7.72	5294	±21	35.456	187	3148
6-56	33-4	2.10	52	"	1.46	20.20	21.21	7.45	5678	±27	34.243	168	2573
7-56	33-9	2.20	49	"	1.73	20.54	24.15	7.25	5774	±3	34.095	166	2314
7-56	33-7	2.40	49	"	2.30	22.40	25.01	6.95	5115	±5	36.977	212	2480
7-56	33-5	2.60	49	"	2.92	24.27	21.30	6.72	3818	±10	44.122	360	3590



Buckling and Critical Mass vs. H₂O/U Ratio
 (1.44% U-235 - 1.336 Diameter)

EXponential MEASUREMENTS OF 1.44 PERCENT ENRICHED URANIUM^{11,12}
1.37 O.D., 0.48" I.D.

Date	Exp. No.	Lattice Spacing Inches	h (cm)	No. Tubes	Fuel Core Cond.	Al/U (vol)	H ₂ O/U (vol)	R _{eff} (cm)	b ₁₁ (cm)	λ (cm)	Buckling (10 ⁻⁶ cm ⁻²)	Std. Error (cm)	R _s (cm)	Vol _s (liters)	CM _s (Lbs)
4-56	32-6	2.00	101.60	49	Wet	0.172	1.51	18.67	20.28	7.44	6006	± 7	33.10	152	2364
4-56	32-1	2.00	81.28	"	Dry	"	1.31	18.67	17.76	7.61	5203	± 5	35.94	195	3025
4-56	32-7	2.10	101.60	"	Wet	"	1.79	19.60	22.72	7.23	6097	± 7	33.01	151	2126
4-56	32-10	2.10	"	"	Dry	"	1.59	19.60	21.09	7.37	5656	± 6	34.40	171	2407
4-56	32-5	2.20	"	45	Wet	"	2.07	19.68	22.18	7.07	6048	± 7	33.33	155	1994
4-56	32-2	2.20	"	"	Dry	"	1.87	19.68	20.79	7.17	5708	± 6	34.41	171	2195
4-56	32-4	2.40	"	49	Wet	"	2.69	22.40	25.49	6.81	5237	± 6	36.60	205	2219
4-56	32-3	2.40	"	43	Dry	"	2.49	20.99	20.81	6.89	5131	± 14	36.97	212	2286
4-56	32-8	2.60	"	49	Wet	"	3.36	24.27	20.57	6.63	3696	± 8	45.05	383	3525
4-56	32-9	2.60	"	"	Dry	"	3.16	24.27	21.09	6.66	3797	± 15	44.33	365	3358



Buckling and Critical Mass vs. H_2O/U Ratio
 (1.44% U-235 - 1.37" O.D. - 0.48" I.D.)

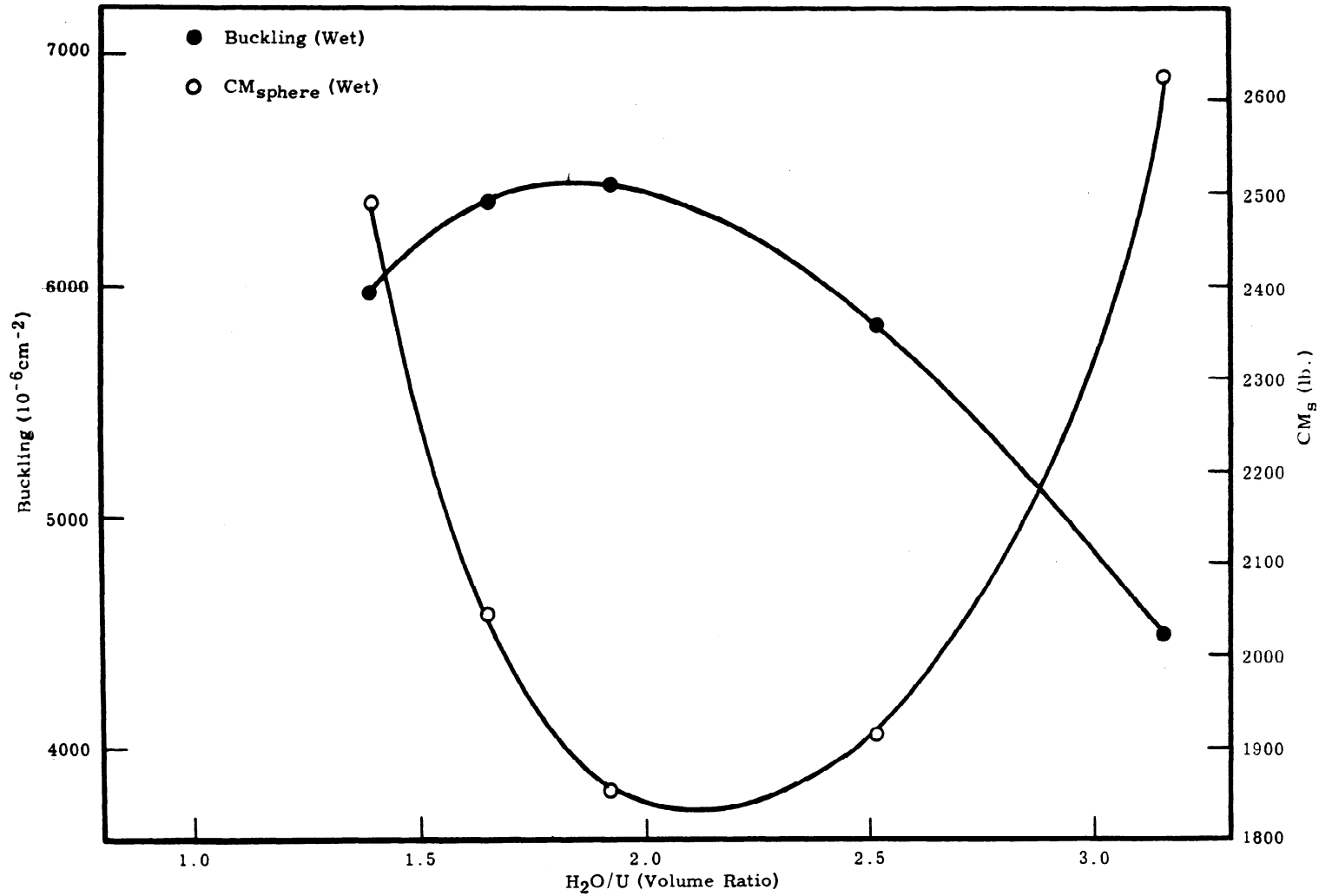
EXPONENTIAL MEASUREMENTS OF 1.44 PER CENT ENRICHED URANIUM¹²

Bismuth in Core of 1.37" O.D. 0.48" I.D., 40" Length

Date	Exp. No.	Lattice Spacing (inches)	No. Tubes	Fuel Core Cond.	Al/U (vol)	H ₂ O/U (vol)	R _{eff} (cm)	b ₁₁ (cm)	λ (cm)	Buckling (10 ⁻⁶ cm ⁻²)	Std. Error	R _s (cm)	Vol _s (Liters)	CM _s (lbs)
11-56	32-16	2.00	49	Dry	.173	1.312	18.67	18.06	7.61	5308	± 8	35.51	187.5	2917
11-56	32-15	2.00	49	B1	"	1.312	18.67	17.90	7.61	5253	± 12	35.74	191.2	2974
11-56	32-14	2.20	45	B1	"	1.875	19.68	20.96	7.17	5746	± 8	34.28	168.7	2169
11-56	32-13	2.20	49	B1	"	1.875	20.54	23.56	7.17	5730	± 5	34.33	169.5	2179
11-56	32-12	2.20	49	Dry	"	1.875	20.54	23.63	7.17	5741	± 5	34.29	168.9	2172

EXPONENTIAL MEASUREMENTS OF 1.466 PERCENT ENRICHED URANIUM¹³
1.394" O.D., 0.464" I.D., 40" Length

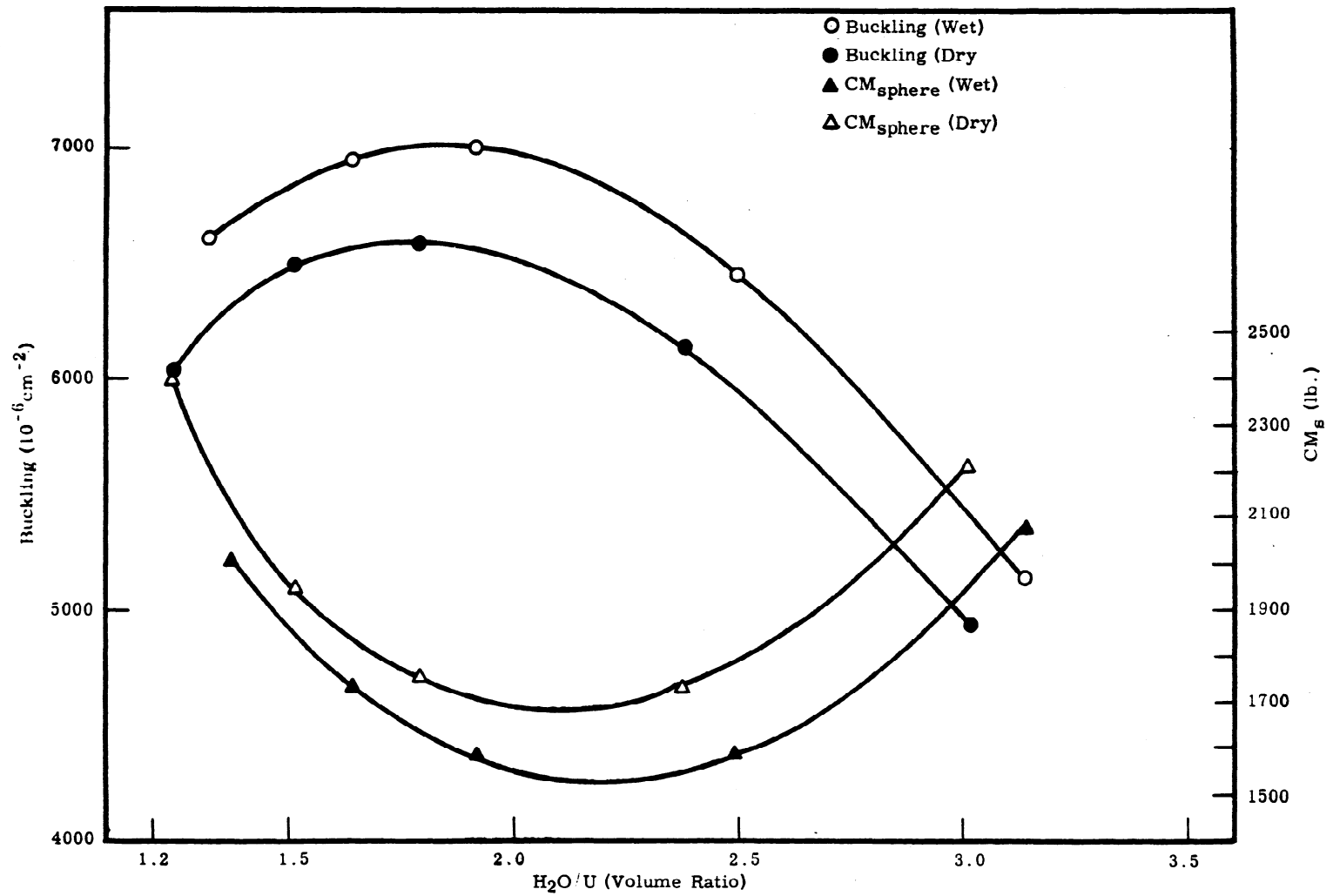
Date	Exp. No.	Lattice Spacing Inches	No. Tubes	Fuel Core Cond.	Al/U (vol)	H ₂ O/U (Vol)	R _{eff} (cm)	b ₁₁ (cm)	λ (cm)	Buckling (10 ⁻⁶ cm ⁻²)	Std. Error	R _s (cm)	Vol _s (liters)	CM _s (lbs)
4-57	37-7	2.00	49	Wet	0.172	1.39	18.67	20.29	7.52	6003	± 14	33.03	151	2464
4-57	37-6	2.00	31	"	"	"	14.85	13.27	"	5876	± 20	33.46	157	2563
4-57	37-5	2.00	43	"	"	"	17.49	17.38	"	5936	± 10	33.26	154	2515
4-57	37-4	2.00	55	"	"	"	19.78	23.65	"	5971	± 4	33.14	152	2488
5-57	37-14	2.10	55	"	"	1.65	20.77	32.13	7.32	6360	± 5	32.07	138	2046
4-57	37-3	2.20	31	"	"	1.92	16.34	15.32	7.14	6229	± 15	32.66	146	1969
3-57	37-2	2.20	43	"	"	"	19.24	22.35	"	6307	± 3	32.42	143	1925
3-57	37-2	2.20	55	"	"	"	21.76	45.38	"	6438	± 3	32.01	137	1854
5-57	37-13	2.40	49	"	"	2.51	22.40	29.30	6.86	5590	± 6	35.16	182	2064
5-57	37-12	2.40	43	"	"	"	20.99	23.63	"	5666	± 7	34.88	178	2014
5-57	37-11	2.40	55	"	"	"	23.74	53.62	"	5828	± 4	34.29	169	1915
5-57	37-10	2.60	31	"	"	3.15	19.31	15.32	6.67	4306	± 19	41.21	293	2831
4-57	37-9	2.60	43	"	"	"	22.74	20.46	"	4296	± 12	41.26	294	2843
4-57	37-8	2.60	55	"	"	"	25.71	31.31	"	4495	± 21	40.19	272	2626



Buckling and Critical Mass vs. H₂O/U Ratio
 (1.466% U-235 - 1.394" O.D. - 0.464" I.D.)

EXPONENTIAL MEASUREMENTS OF 1.6 PERCENT ENRICHED URANIUM¹⁴
1.394" O.D., 0.464" I.D., 40" Length

Date	Exp. No.	Lattice Spacing (inches)	No. Tubes	Fuel Core Cond.	Al/U (vol)	H ₂ O/U (vol)	R _{eff} (cm)	b ₁₁ (cm)	λ (cm)	Buckling (10 ⁻⁶ cm ⁻²)	Std. Error	R _S (cm)	Vol _S (liters)	CM _S (lbs)
3-58	58	2.00	43	Wet	.165	1.375	17.49	19.49	7.53	6605	± 7	31.13	126	2055
4-58	59	2.00	43	Dry	"	1.25	17.49	17.91	7.65	6033	± 14	32.80	148	2405
3-58	54	2.10	43	Wet	"	1.637	18.36	23.52	7.35	6941	± 4	30.36	117	1730
3-58	53	2.10	43	Dry	"	1.512	18.36	21.33	7.45	6483	± 10	31.57	132	1945
3-58	55	2.20	43	Wet	"	1.91	19.24	27.62	7.14	7000	± 7	30.41	118	1584
3-58	51	2.20	31	Dry	"	1.79	16.34	16.21	7.25	6587	± 45	31.46	130	1754
6-58	69	2.20	55	"	"	1.79	21.76	59.81	7.25	6592	± 4	31.44	130	1751
3-58	52	2.20	43	"	"	1.79	19.24	24.59	7.25	6587	± 13	31.46	130	1754
3-58	56	2.40	43	Wet	"	2.497	20.99	31.70	6.88	6450	± 4	32.24	140	1586
4-58	61	2.40	43	Dry	"	2.373	20.99	27.92	6.93	6136	± 10	33.18	153	1728
3-58	57	2.60	43	Wet	"	3.135	22.74	25.35	6.68	5125	± 10	37.21	216	2078
4-58	60	2.60	43	Dry	"	3.011	22.74	24.00	6.70	4937	± 2	38.01	230	2216



Buckling and Critical Mass vs. H_2O/U Ratio
 (1.6% U-235 - 1.394" O.D. - 0.464" I.D.)

CRITICAL APPROACH MEASUREMENTS OF 1.6 PER CENT ENRICHED URANIUM¹⁵

(1.394" O.D., 0.464" I.D., 40" Length)

Date	Exp. No.	Lattice Spacing (inches)	H ₂ O/U (vol)	Ext. N Tubes	CM cyl. (lbs)	R _c (cm)	λ (cm)	Buckling (10 ⁻⁶ cm ⁻²)	R _s (cm)	Vol _s (liters)	CM _s (lbs)
7-58	0-1	2.20	1.91	60.58 ± .86	2221 ± 32	22.745	7.14	7210	29.858	111.5	1499
7-58	0-1a	2.20	1.91	59.09 ± .12	2183 ± 44	22.553	7.14	7294	29.644	109.1	1468
6-58	0-2	2.20	1.79	68.08 ± .78	2516 ± 29	24.208	7.25	6576	31.491	130.8	1759
6-58	0-2a	2.20	1.79	64.21 ± .47	2373 ± 17	23.510	7.25	6844	30.724	121.5	1634
6-58	0-3	2.40	2.50	57.43 ± 1.85	2122 ± 68	24.255	6.88	6707	31.480	130.7	1476
6-58	0-3a	2.40	2.50	58.11 ± 0.47	2147 ± 17	24.398	6.88	6653	31.636	132.6	1499
6-58	0-4	2.40	2.37	64.86 ± .38	2397 ± 14	25.776	6.93	6147	33.141	152.4	1723
6-58	0-4a	2.40	2.37	62.31 ± 1.2	2302 ± 44	25.265	6.93	6320	32.589	145.0	1638

The Al/U Volume Ratio for all above exp. was .165.

Type of plot to get Ext. N was N/CR.

Poison corrections were calculated for exp. 0-1. These amounted to 0.48 rod so were not made for others.

Exp. No. with (a) denotes back off, the others are approach measurements.

Variation in H₂O/U for the same lattice spacing is due to dry or wet center core.

EXPONENTIAL MEASUREMENTS OF 3.063 PER CENT ENRICHED URANIUM^{17,18,19,20,21,22}

Date	Exp. No.	Slug Dia. (inches)	Lattice Spacing (inches)	H ₂ O/U (volume)	Height, h (cm)	No. Tubes	R _{eff} (cm)	Relaxation Length, b ₁₁ (cm)
11-58	12-c	0.925	1.950	3.89	81.28	31	14.48	23.67
11-58	12-b	"	"	"	60.96	"	"	24.91
6-59	42	0.600	1.000	2.06	81.28	97	13.13	39.24
6-59	44	"	1.100	2.71	"	73	12.53	30.40
7-59	46	"	1.200	3.41	"	67	13.10	35.66
7-59	47	"	1.300	4.18	"	61	13.54	35.07
7-59	50	"	1.420	5.18	"	61	14.79	41.10
7-59	52	"	1.600	6.84	"	65	17.20	37.87
3-59	21	0.300	0.500	2.06	81.28	289-1	11.34	15.88
5-59	35	"	0.600	3.41	"	253-1	12.73	43.87
4-59	23	"	0.700	5.00	"	187	12.77	32.38
4-59	25	"	0.800	6.84	"	163	13.62	31.58
5-59	27	"	0.900	8.92	"	163	15.32	35.08
12-59	71	0.175	0.300	2.24	59.69	781-1	11.18	14.01
1-60	74	"	0.375	4.06	"	611-1	12.86	38.40
10-59	61	"	0.450	6.29	"	433-1	12.49	24.27
10-59	57	"	0.500	8.00	"	279-1	12.98	23.00
10-59	59	"	0.550	9.89	"	379	14.28	27.17
10-59	63	"	0.600	11.96	"	379	15.58	28.18

CRITICAL APPROACH MEASUREMENTS OF 3.063 PERCENT ENRICHED URANIUM ^{17,18,19,20,21,22}

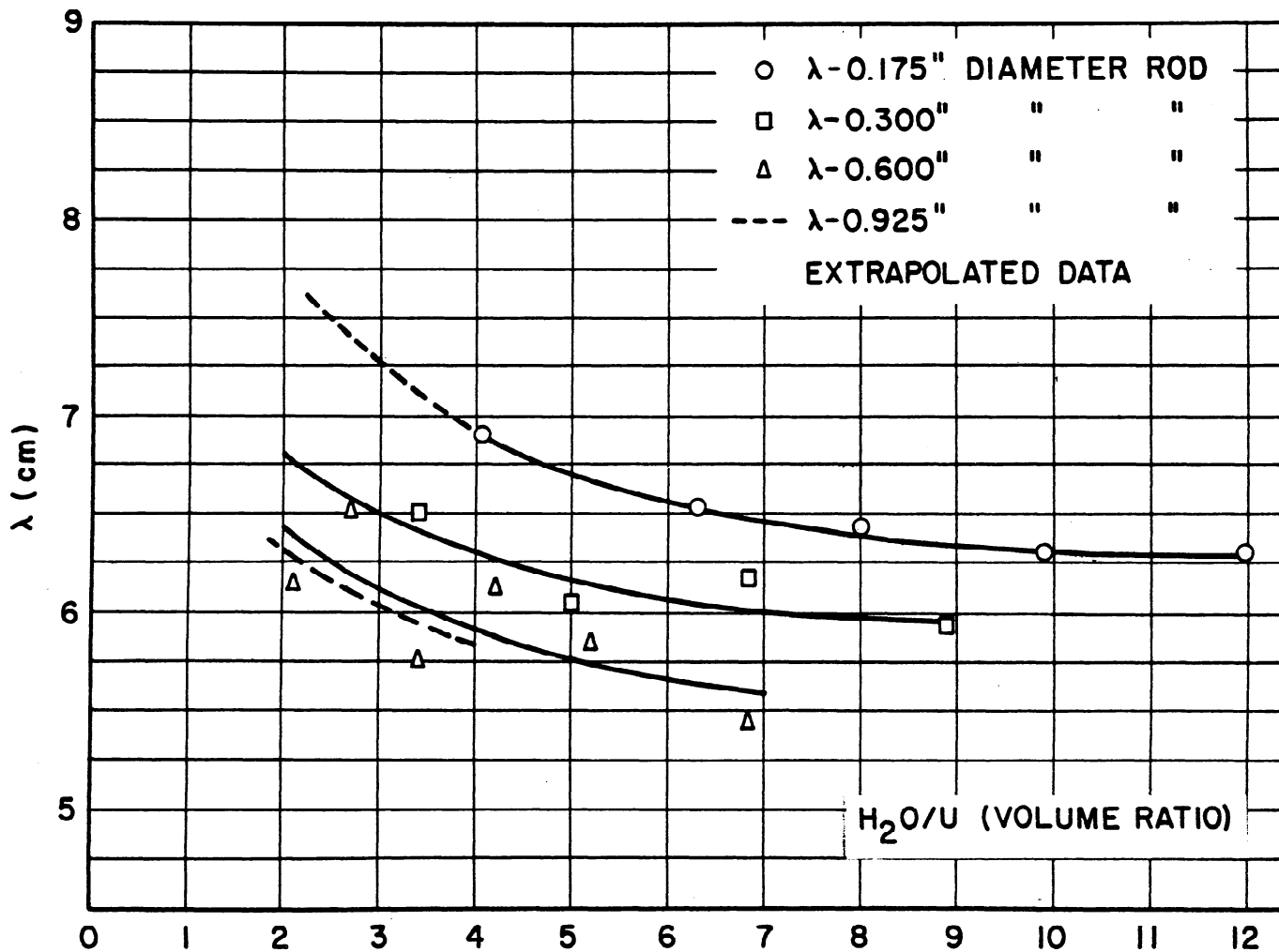
Date	Exp. No.	Slug Dia. (inches)	Lattice Spacing (inches)	h (cm)	H ₂ O/U (Vol)	N ⁽¹⁾ (rods)	R _c (cm)	C.M. Cylinder (lb)	λ ⁽²⁾ (cm)	Buckling (10 ⁻⁶ cm ⁻²)	C.M. Sphere (lb)
9-58	8	0.925	1.500	40.64	1.89	72.08	16.98	530.7	6.37	14068	497
7-58	2	"	"	60.96	"	60.0	15.50	662.6			
8-58	4	"	1.600	40.64	2.29	63.5	17.01	467.6	6.23	14212	437
7-58	3	"	"	60.96	"	52.4	15.45	578.8			
8-58	5	"	1.700	40.64	2.72	58.6	17.35	431.3	6.12	14027	403
7-58	1	"	"	60.96	"	49.2	15.90	543.1			
9-58	6	"	1.800	40.64	3.17	56.7	18.08	417.6	6.00	13536	392
9-58	7	"	"	60.96	"	46.9	16.44	518.1			
10-58	11	"	1.950	40.64	3.89	60.4	20.21	444.7	5.86	12108	421
11-58	12a	"	"	60.96	"	48.6	18.13	536.8			
12-58	15	0.600	1.000	40.64	2.06	152.1	16.45	471.2	6.16	14834	433
6-59	41	"	"	81.28	"	114.9	14.30	711.9			
10-58	10	"	1.100	40.64	2.71	118.6	15.97	367.4	6.52	14855	337
6-59	43	"	"	81.28	"	91.2	14.01	564.8			
9-58	9	"	1.200	40.64	3.41	104.5	16.36	323.7	5.77	15435	295
7-29	45	"	"	81.28	"	79.9	14.30	494.9			
12-58	13	"	1.300	40.64	4.18	98.9	17.24	306.4	6.13	14115	283
7-59	48	"	"	81.28	"	74.1	14.92	458.8			
12-58	14	"	1.420	40.64	5.18	100.3	18.97	310.7	5.86	12984	290
7-59	49	"	"	81.28	"	73.8	16.27	457.1			
12-58	16	"	1.600	40.64	6.84	122.0	23.57	378.0	5.46	10576	353
7-59	51	"	"	81.28	"	82.4	19.37	510.4			
3) 3-59	21	0.300	0.500	81.28	2.06	624	16.7	964	6.80	13607	465
2-59	19	"	0.600	40.64	3.41	387.5	15.75	299.2	6.51	15098	274
5-59	33	"	"	81.28	"	298.1	13.82	460.3			
2-59	18	"	0.700	40.64	5.00	296.9	16.09	229.2	6.05	15352	209
4-59	22	"	"	81.28	"	230.0	14.16	355.3			
2-59	17	"	0.800	40.64	6.84	271.9	17.59	209.9	6.17	13758	195
6-59	40	"	"	81.28	"	199.9	15.09	308.7			
2-59	20	"	0.900	40.64	8.92	285.7	20.29	220.6	5.94	11986	207
4-59	26	"	"	81.28	"	203.8	17.14	314.7			
3) 12-59	71	0.175	0.300	59.69	2.24	1786	16.9	689	7.5	11477	560
12-59	72	"	0.375	59.69	4.06	873.3	14.78	337.1	6.91	14121	257
10-59	60	"	0.450	59.69	6.29	628.3	15.04	242.5	6.53	14292	184
10-59	56	"	0.500	59.69	8.00	569.7	15.92	219.9	6.43	13458	171
10-59	58	"	0.550	59.69	9.89	554.2	17.27	213.9	6.30	12301	171
10-59	62	"	0.600	59.69	11.96	572.5	19.15	221.0	6.31	10809	184

(1) N/CR vs N used to evaluate N_c.

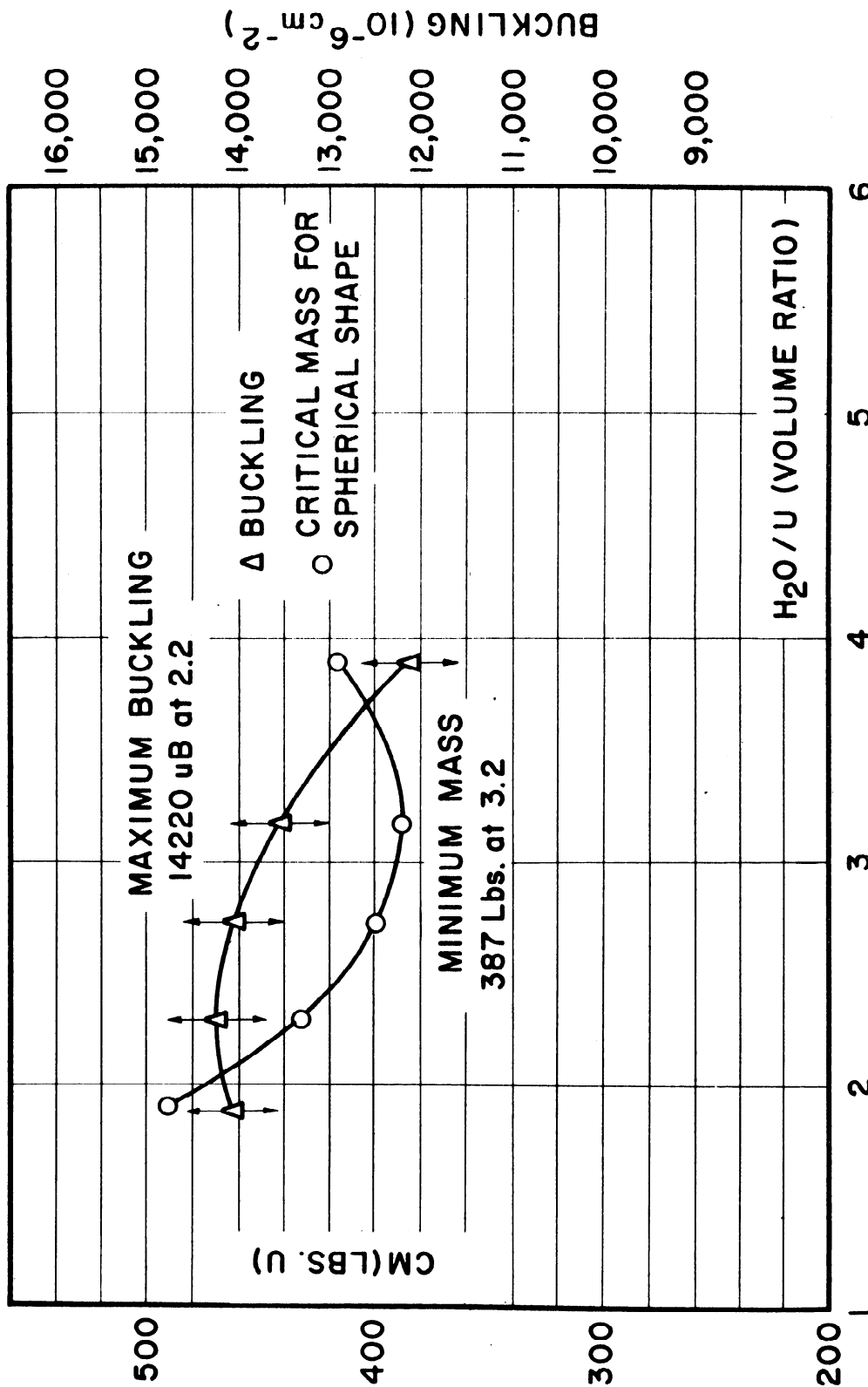
(2) λ for 0.925 diameter extrapolated from curves of λ vs rod diameter. λ calculated from setting values of buckling equal, when more than two measurements were made, a weighted average was taken.

(3) Exponential measurement only - λ from extrapolating measured λ curve.

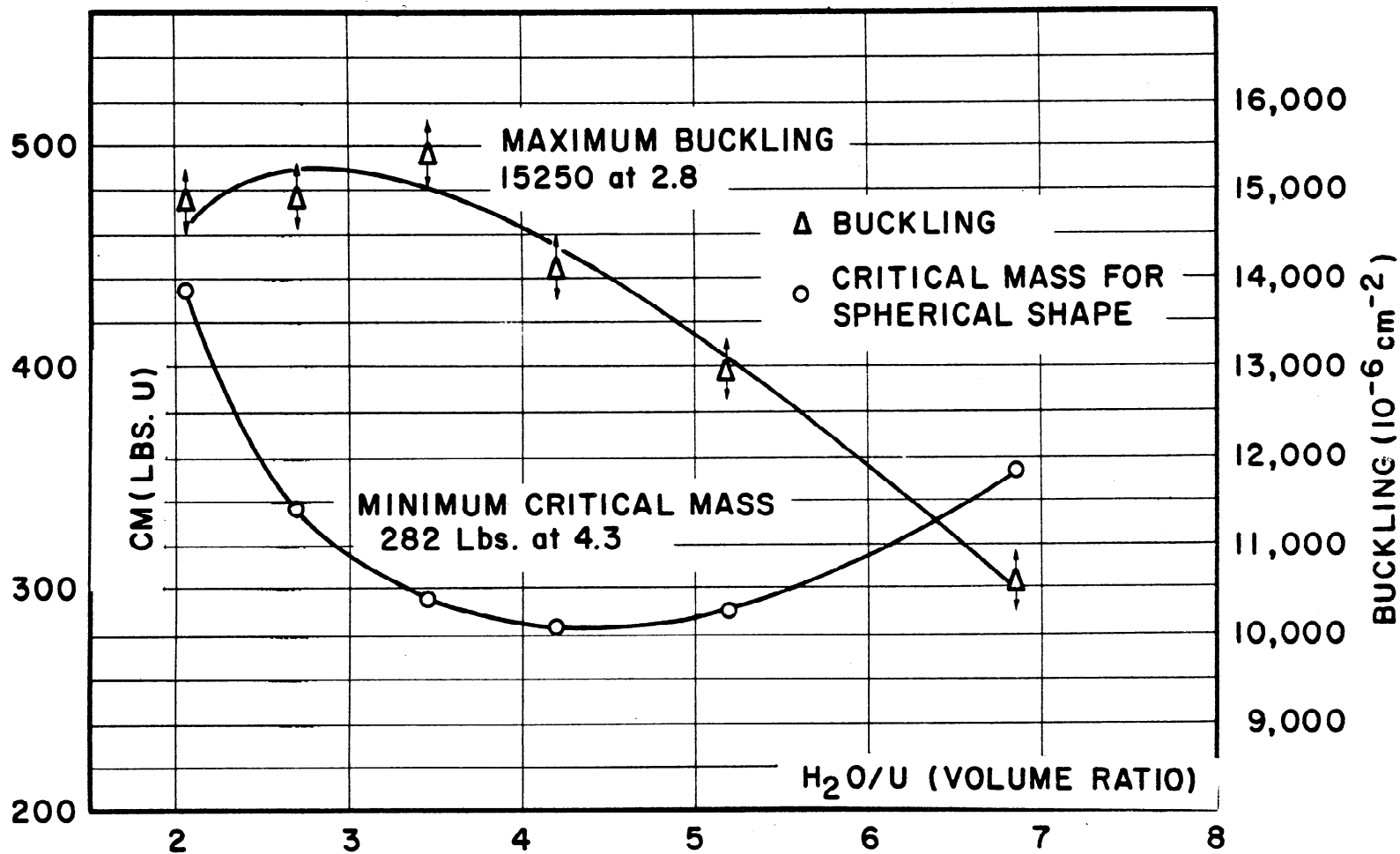
**EXTRAPOLATION LENGTH VS. H₂O/U VOLUME RATIO
(for 3.063 Per Cent Enriched Uranium)**



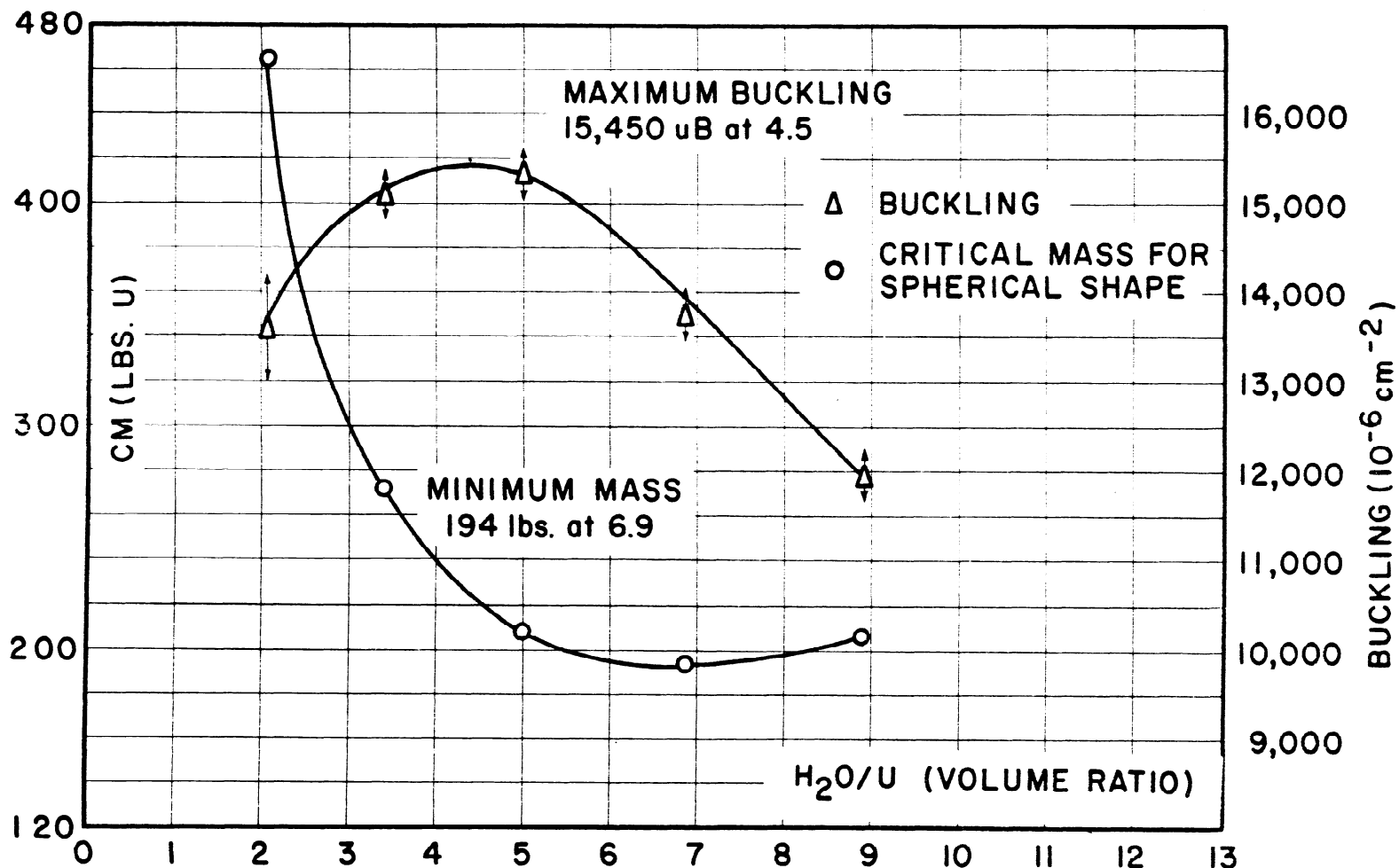
**CRITICAL MASS AND BUCKLING vs. H₂O/U VOLUME RATIO
(3.063 PER CENT ENRICHED URANIUM, 0.925" DIA. RODS)**



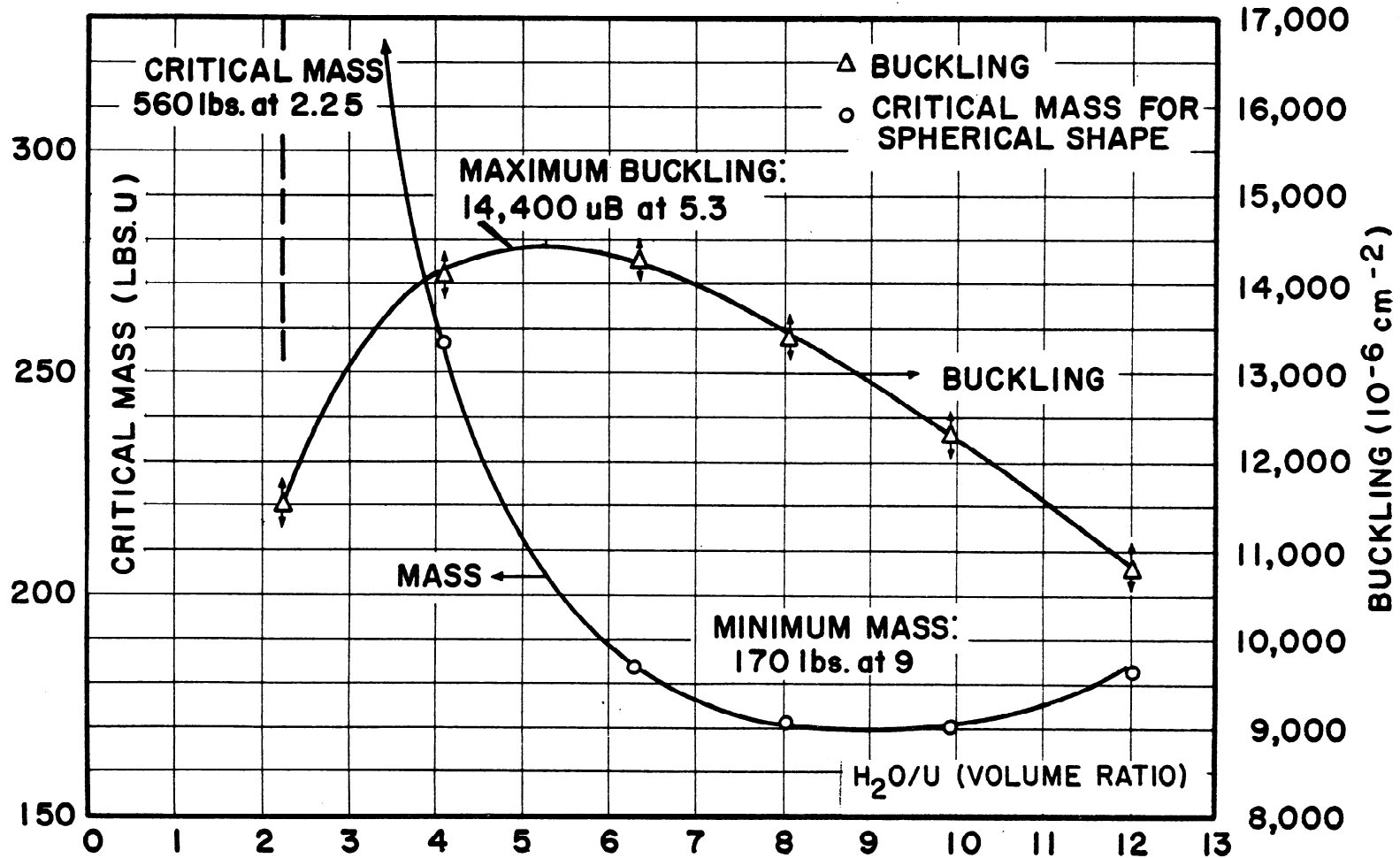
**CRITICAL MASS AND BUCKLING vs. H₂O/U VOLUME RATIO
(3.063 PER CENT ENRICHED URANIUM , 0.600" DIA. RODS)**



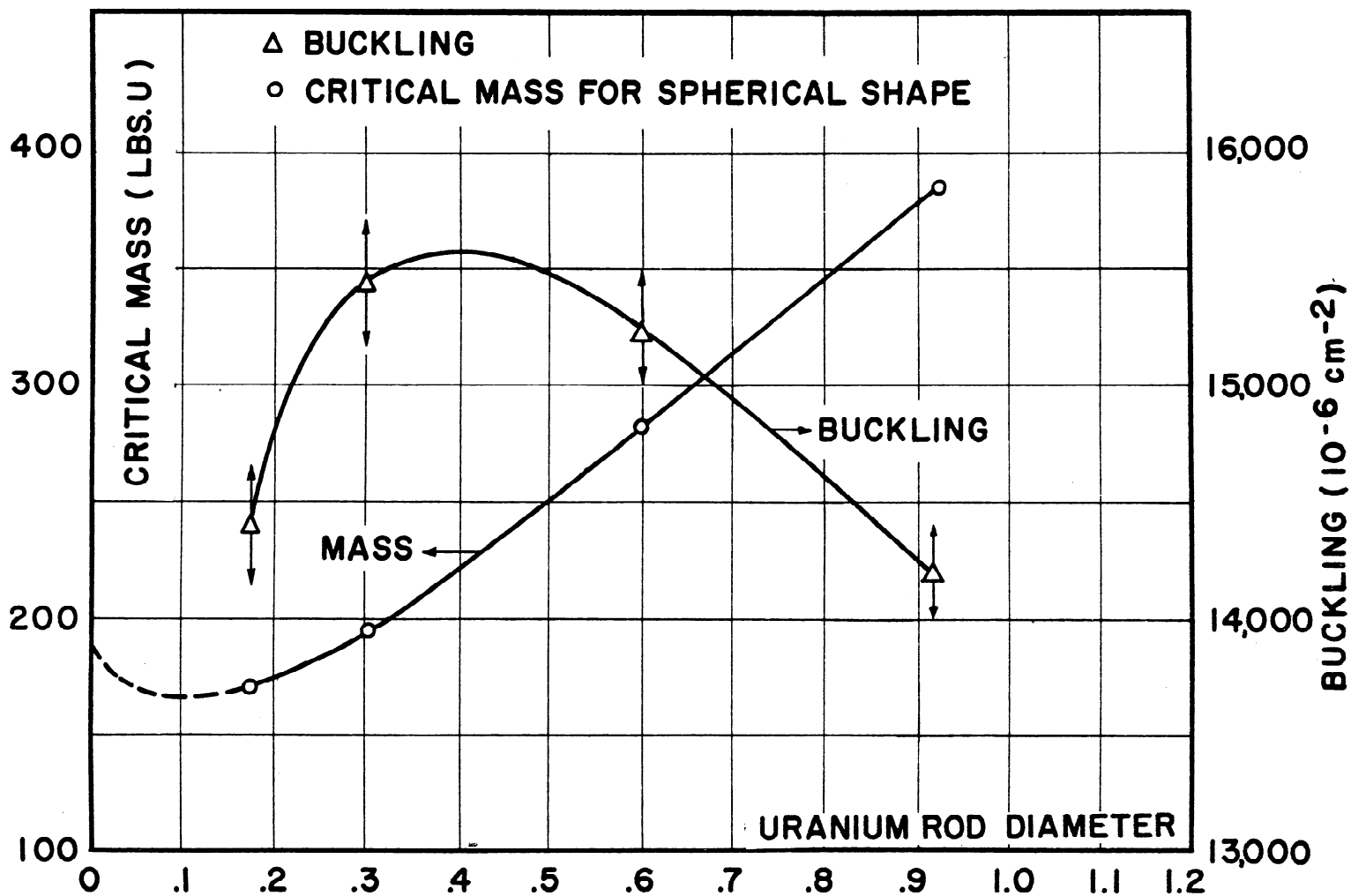
**CRITICAL MASS AND BUCKLING vs. H₂O/U VOLUME RATIO
(3.063 PER CENT ENRICHED URANIUM, 0.300" DIA. RODS)**



**CRITICAL MASS AND BUCKLING VS. H₂O/U VOLUME RATIO
(3.06 Per cent Enriched Uranium, 0.175-Inch Dia. Rods)**



MAXIMUM BUCKLING AND MINIMUM MASS VS. ROD DIAMETER (3.06 Percent Enriched Uranium)



SHAPE PERTURBATIONS OF 3.063 PERCENT ENRICHED URANIUM^(23,24,25)
 (0.175" Diameter, 23.5" Length, 0.50" Lattice Spacing)

ELLIPTIC CYLINDERS

Date	Exp. No.	Eccentricity	Axis		N_c (rods)	CM cyl. (lb.)	B^2 (10^{-6}cm^{-2})	λ
			Semi-Major	Semi-Minor				
11-19-59	69		15.88	15.88	567.3 ± 1	219.0 ± 0.4	13,530	6.4
11-9-59	64	.212 *(.224)	16.07 16.2	15.71 15.6	568.2 ± 1.5	219.3 ± 0.6	13,520	6.4
11-11-59	65	.465 (.480)	16.92	14.98	570.3 ± 2.5	220.1 ± 1.0	13,520	6.4
11-13-59	6.6	.647 (.624) (.74)	18.34 19.2 21.9	13.99 13.6 12.6	572.2 ± 1.5	222.8 ± 0.6	13,550	6.4

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*Calculated values from (25), others calculated from measured moments.

RECTANUGLAR CYLINDERS

Date	Exp. No.	Critical Dimensions		N_c	CM cyl.	B^2	λ
		a (cm)	b (cm)	(rods)	(lb)	(10^{-6}cm^{-2})	(cm)
11-16-59	67	28.09	29.22	587.7 ± 0.6	226.9 ± 0.2	13,440	6.3
11-17-59	68	25.30	33.00	597.7 ± 1.5	230.7 ± 0.6	13,510	6.3

REFERENCES

1. Ketzlach, N., and Lloyd, R. C., "Nuclear Safety of Iron Encased Fuel Elements", Nuclear Physics Research Quarterly Report, April, May, June, 1958, HW-56919, pp. 27-28.
2. Ketzlach, N., Nuclear Safety of Iron Encased Fuel Elements, HW-56423, June, 1958.
3. Clayton, E. D., Exponential Pile Measurements in Water Moderated Lattices with Enriched Uranium Rods, HW-40930, January, 1956.
4. Clayton, E. D., "Water Moderated Lattices with Enriched Uranium Rods", Nuclear Physics Research Quarterly Report, October, November, December, 1955, HW-42182.
5. Bennett, R. A., E. Z. Block, "Reflector Savings of One Per Cent Enriched Uranyl Nitrate", Nuclear Physics Research Quarterly Report, October, November, December, 1957, HW-54591, pp. 43-63.
6. Lloyd, R. C., "Buckling Measurements of Fuel Elements in a Random Array, Water Moderated", Nuclear Physics Research Quarterly Report, October, November, December, 1957, pp. 35-36.
7. Block, E. Z., and E. D. Clayton, "Buckling Measurements with Enriched Fuel Elements in Light Water", Nuclear Physics Research Quarterly Report, July, August, September, 1956, HW-47012, pp. 12-17.
8. Lloyd, R. C., "Buckling Measurements for Fuel Elements in A Random Array", Nuclear Physics Research Quarterly Report, January, February, March, 1958, HW-55879, pp. 12-44.
9. Lloyd, R. C., Buckling Measurements of Fuel Elements in Random Arrays, HW-57853, October, 1958.
10. Lloyd, R. C., "Exponential Measurements with 1.25 Per Cent Enriched Uranium", Nuclear Physics Research Quarterly Report, January, February, March, 1959, HW-60220, pp. 55-57.
11. Clayton, E. D., E. Z. Block, "Buckling Measurements with Enriched Fuel Elements in Light Water", Nuclear Physics Research Quarterly Report, April, May, June, 1956, HW-44525, pp. 59-67.
12. Block, E. Z., "Enriched Uranium-Water Lattices", Nuclear Physics Research Quarterly Report, October, November, December, 1956, HW-48893, pp. 43-44.
13. Block, E. Z., "Enriched Uranium-Water Lattices", Nuclear Physics Research Quarterly Report, April, May, June, 1957, HW-51983, pp. 49-54.
14. Lloyd, R. C., "Nuclear Safety Specification for Enriched Uranium in the 1.25-2 Per Cent Range", Nuclear Physics Research Quarterly Report, January, February, March, 1958, HW-55879, pp. 9-12.

15. Lloyd, R. C., E. D. Clayton, H. E. Handler, "Criticality Parameters for Lattices with 1.6 Per Cent Enriched Uranium Rod in Light Water", Nuclear Physics Research Quarterly Report, April, May, June, 1958, HW-56919, pp. 22-24.
16. Lloyd, R. C., E. D. Clayton, R. B. Smith, "Criticality Measurements of Heterogeneous 3.1% Enriched Uranium-Water Systems", Nuclear Physics Research Quarterly Report, July, August, September, 1958, HW-57861.
17. Lloyd, R. C., E. D. Clayton, R. B. Smith, "Criticality Measurements of Heterogeneous 3.1% Enriched Uranium-Water Systems", Nuclear Physics Research Quarterly Report, October, November, December, 1958, HW-59126.
18. Lloyd, R. C., E. D. Clayton, R. B. Smith, "Criticality Measurements of Heterogeneous 3.1% Enriched Uranium-Water Systems", Nuclear Physics Research Quarterly Report, January, February, March, 1959, HW-60220.
19. Lloyd, R. C., E. D. Clayton, R. B. Smith, "Criticality Measurements of Heterogeneous 3.1% Enriched Uranium-Water Systems", Nuclear Physics Research Quarterly Report, April, May, June, 1959, HW-61181.
20. Lloyd, R. C., E. D. Clayton, R. B. Smith, "Criticality Measurements of Heterogeneous 3.1% Enriched Uranium-Water Systems", Nuclear Physics Research Quarterly Report, July, August, September, 1959, HW-62727.
21. Lloyd, R. C., R. B. Smith, E. D. Clayton, Neutron Multiplication Measurements of Heterogeneous 3.1 Per Cent Enriched Uranium-Water Systems, paper presented at the American Nuclear Society Meeting, Gatlinburg, Tennessee, June, 1959.
22. Lloyd, R. C., E. D. Clayton, R. B. Smith, and V. I. Neeley, "Criticality Measurements of Heterogeneous 3.1 Per Cent Enriched Uranium and Water Systems", Nuclear Physics Research Quarterly Report, October, November, December, 1959, HW-63576.
23. Reardon, W. A., R. C. Lloyd, "Shape - Perturbations in Critical Experiments", Nuclear Physics Research Quarterly Report, October, November, December, 1959, HW-63576.
24. Reardon, W. A., R. C. Lloyd, "Shape - Perturbations in Critical Experiments", paper presented at American Nuclear Society Meeting, Chicago, Illinois - June, 1960.
25. Gast, P. F., A. Bournia, "Finding the Buckling of an Elliptic Cylinder", Nucleonics, Vol. 14, No. 4, 1956, pp. 109-113.